

Effect of internal stress on peak position: problems and opportunities

HCP materials

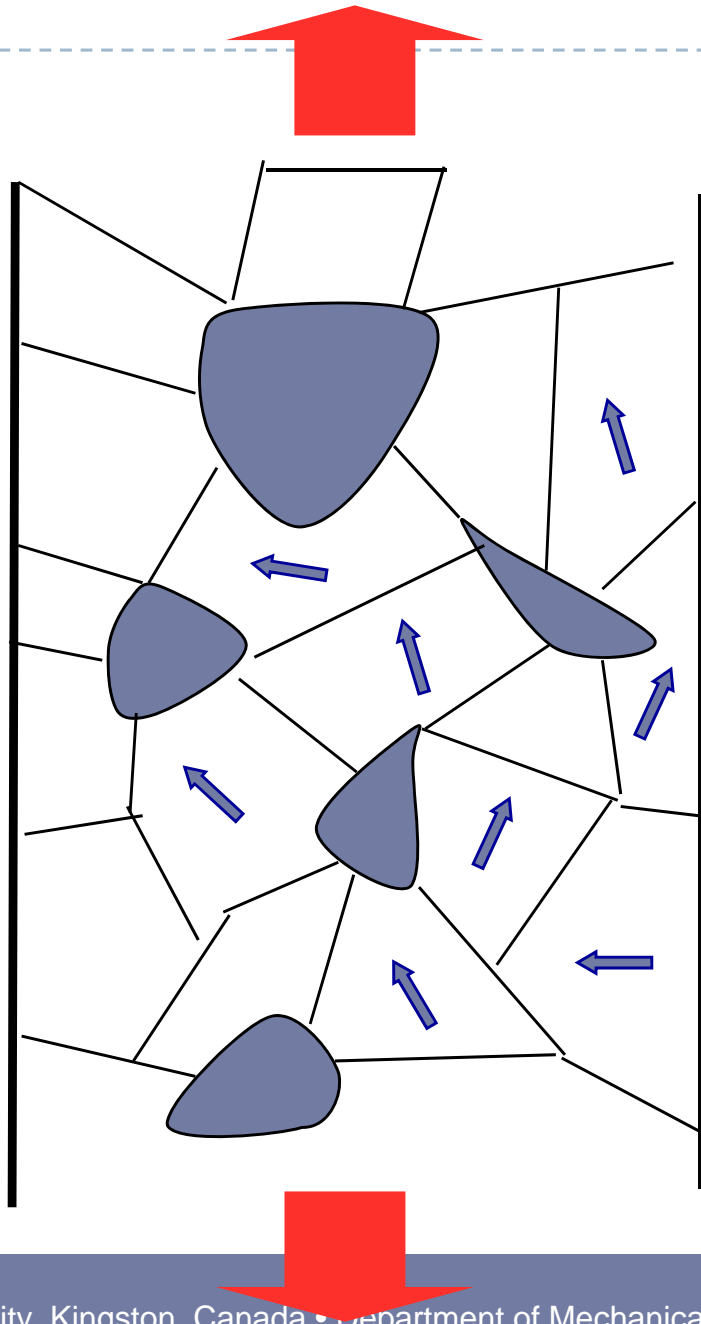
Mark Daymond

APD IV

Outline

- ▶ Interphase and intergranular stresses
- ▶ Effect on Rietveld refinement
- ▶ Opportunities to determine crystal properties
 - ▶ Impact of high energy neutrons on structure
 - ▶ Use of internal stresses to determine operating deformation modes
- ▶ Conclusions

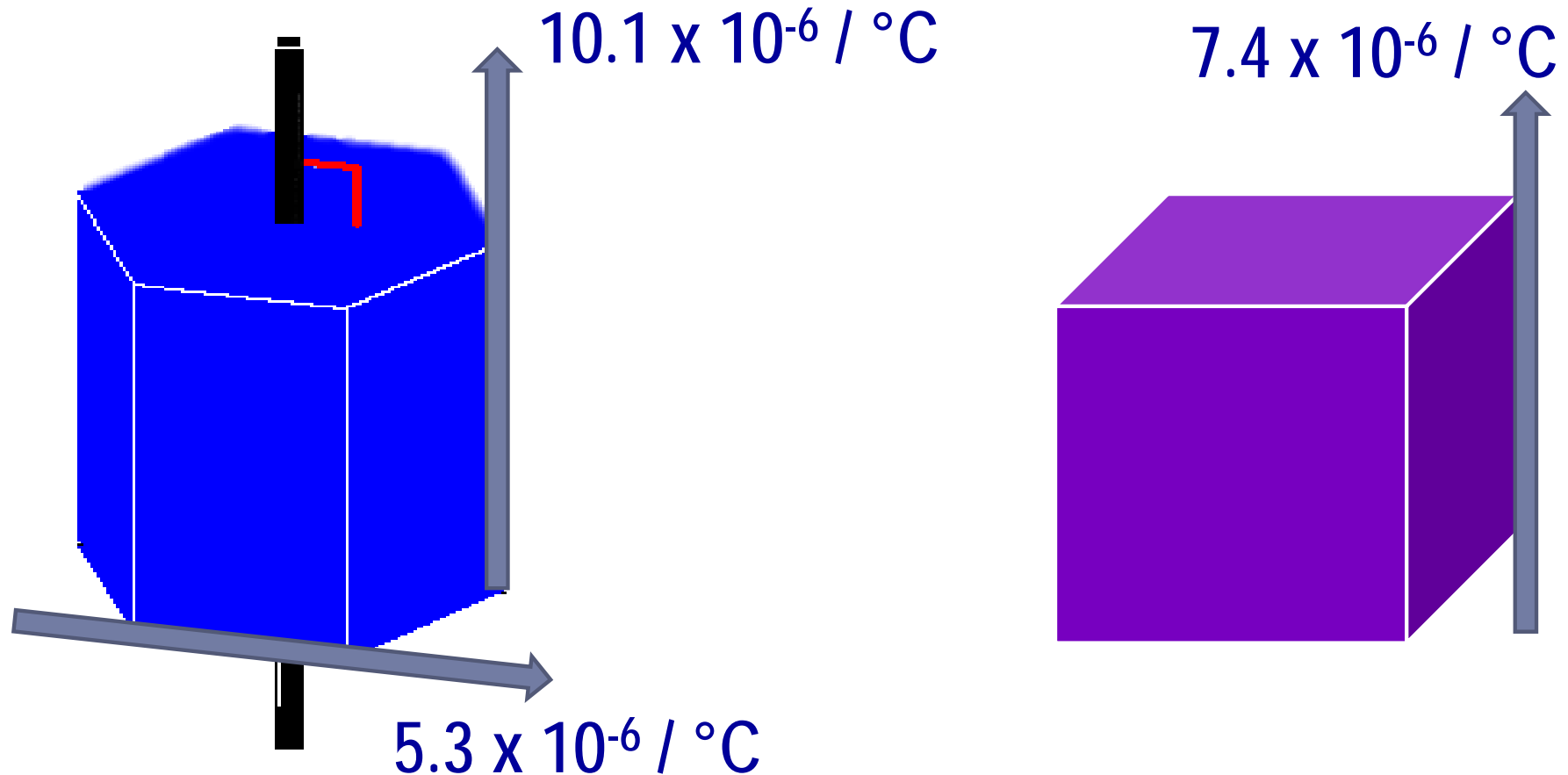
Constraints on phases and grains



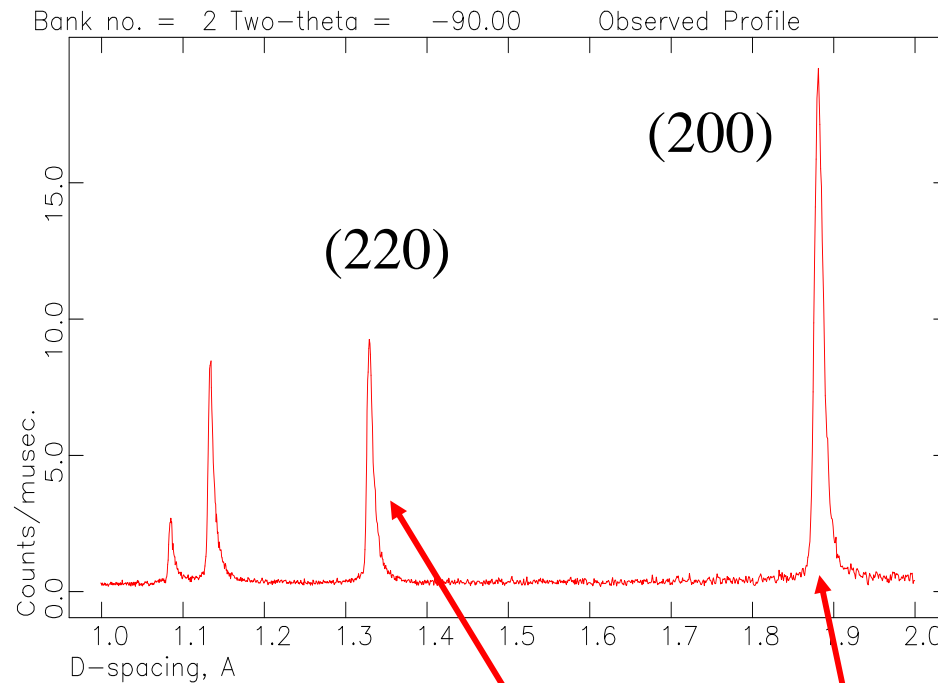
What is impact of interactions between phases and different grain orientations on stress / deformation?

Thermal anisotropy

- ▶ e.g. Zirconium-Niobium alloy; two phase HCP and BCC



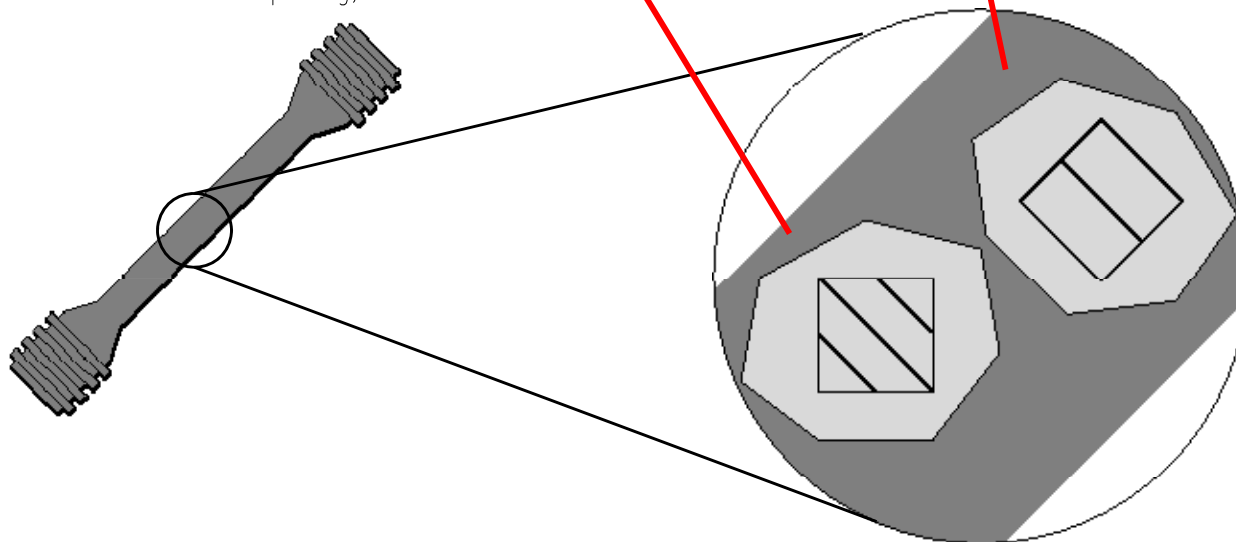
Diffraction measurement of elastic strain



$$\lambda = 2d \sin \theta$$

$$\varepsilon = \Delta d / d$$

Diffraction peaks
measured *in specific
direction in sample*



(cubic example shown)

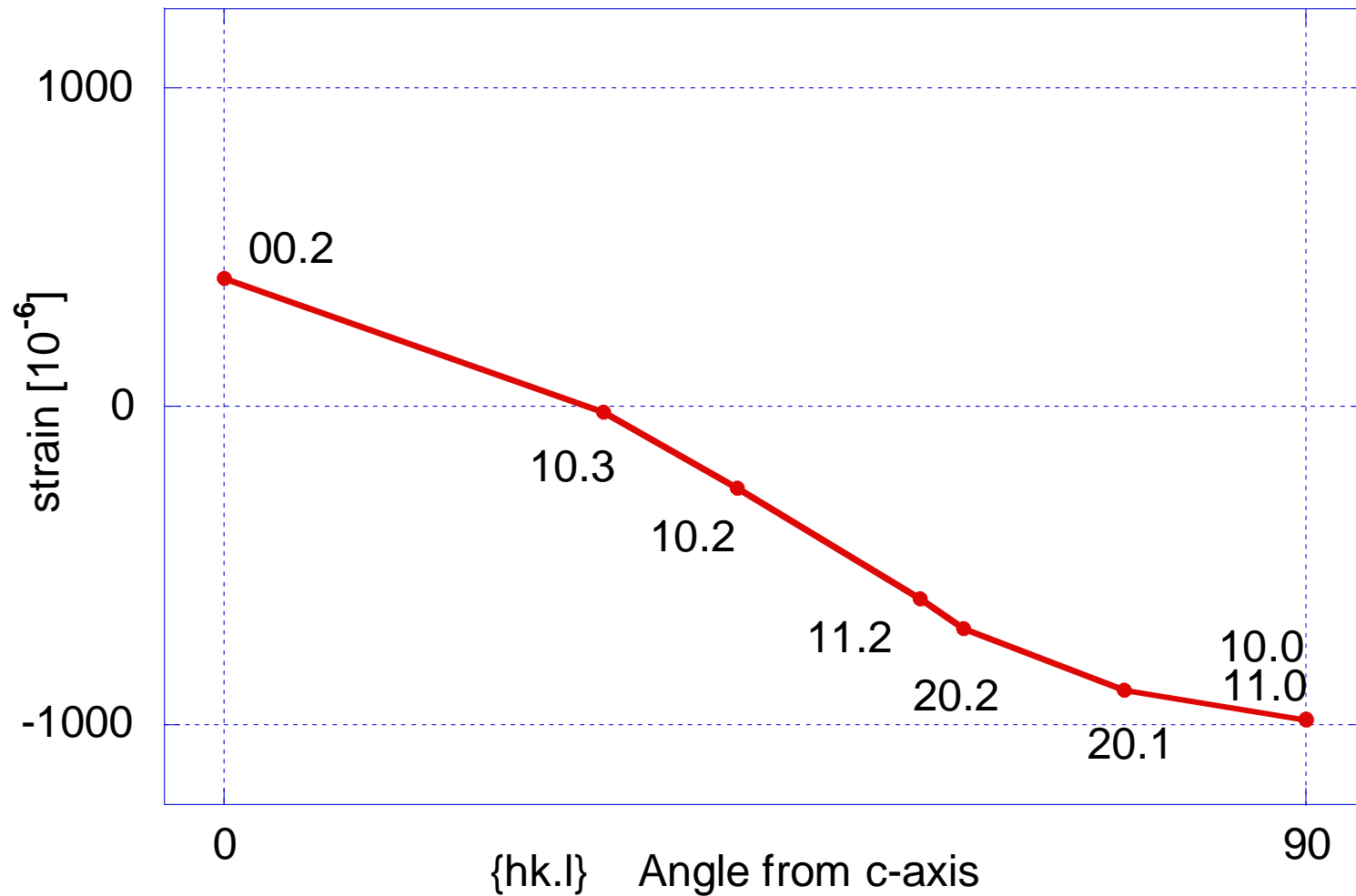
Relationship d - hkl and a & c

$$d_{hkl} = \frac{a}{\sqrt{h^2 + k^2 + l^2}} \quad \text{cubic}$$

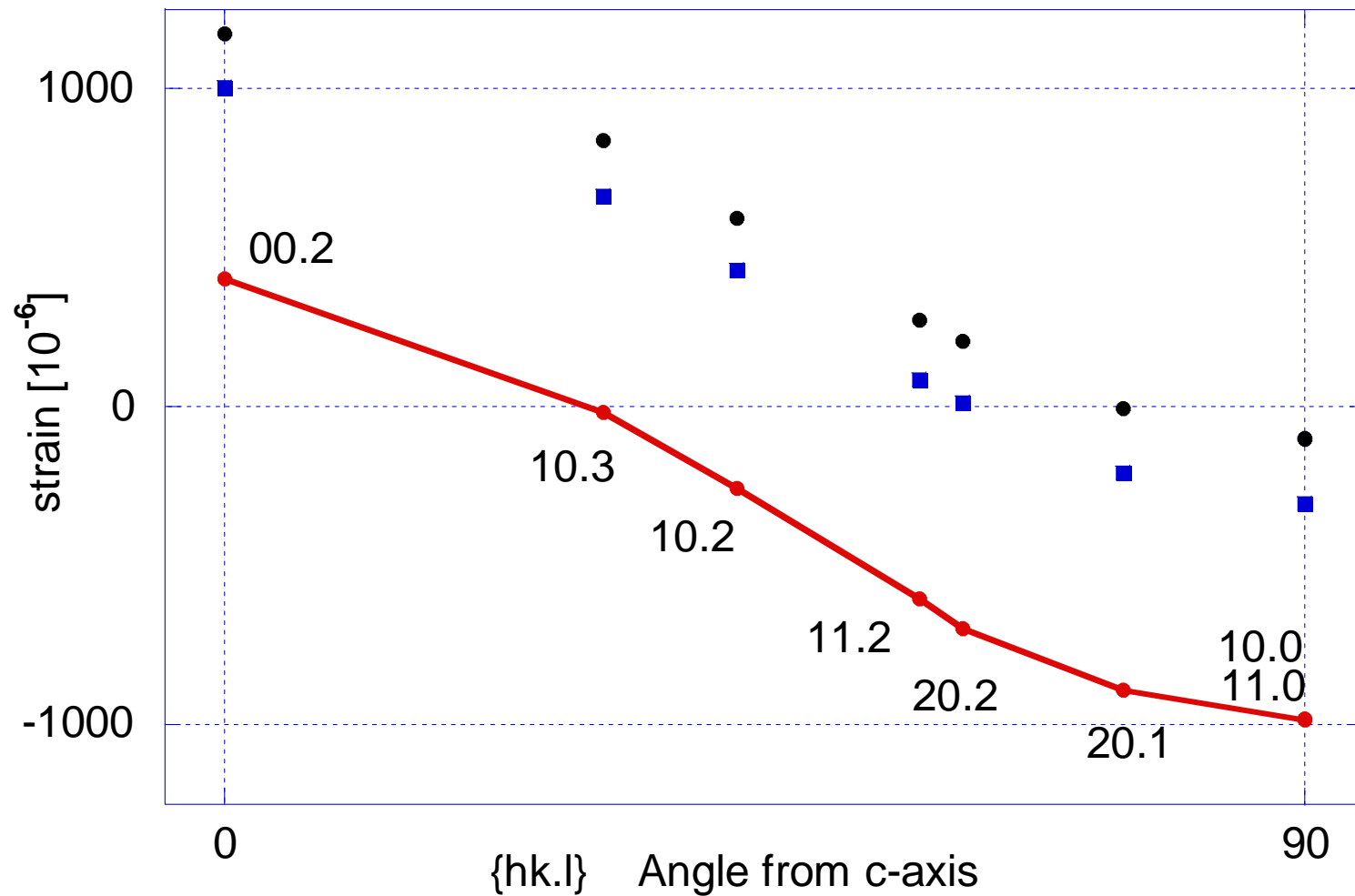
$$\frac{1}{d_{hkl}^2} = \frac{4}{3} \left(\frac{h^2 + hk + k^2}{a^2} \right) + \frac{l^2}{c^2} \quad \text{hexagonal}$$

Will hold for single crystal, a single-crystal powder, and a “stress-free” polycrystal

Deviation from ideal relation due to: thermal stresses



Deviation from ideal relation due to: thermal stresses – effect of texture

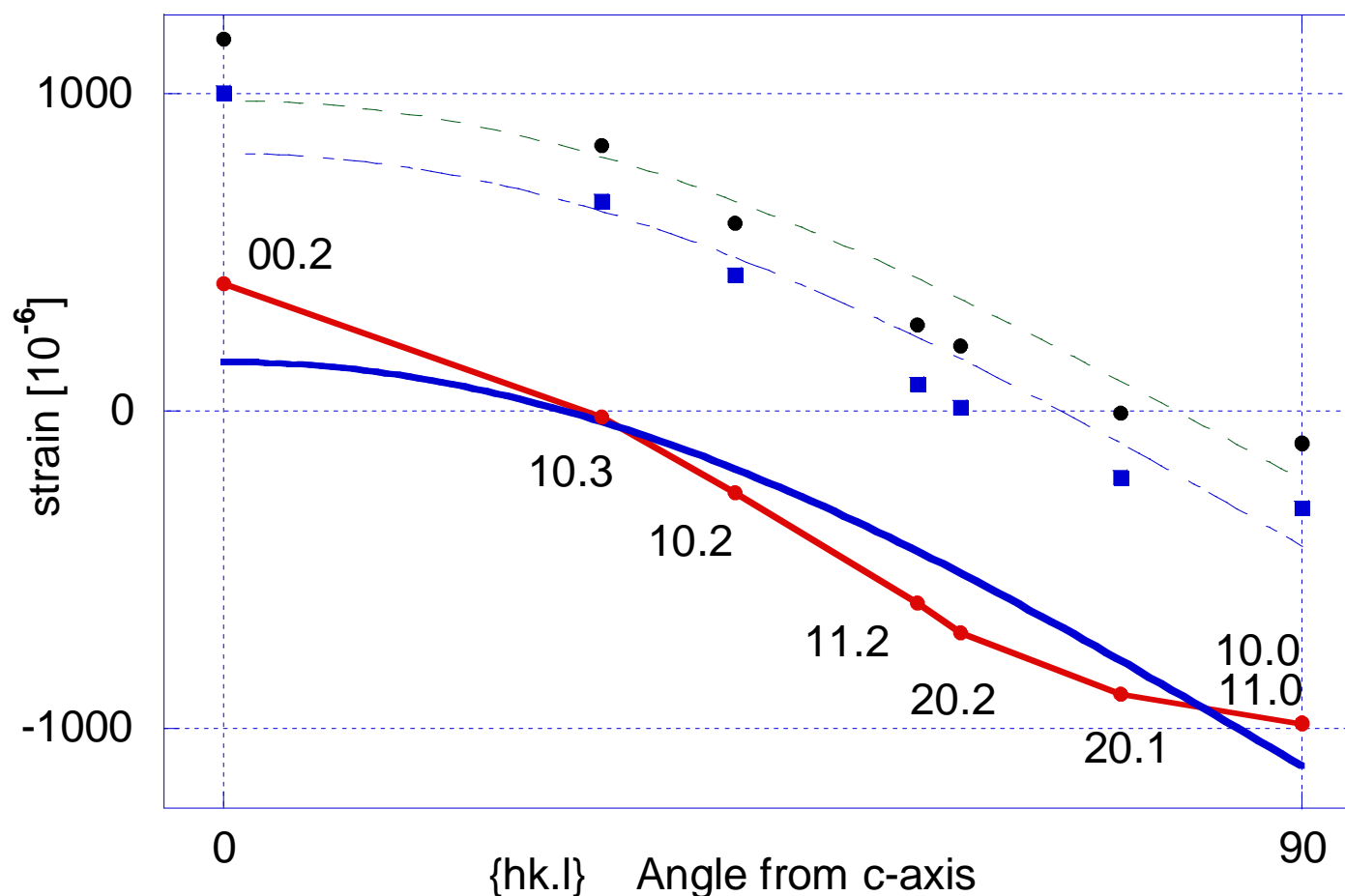


How to handle in Rietveld refinement?

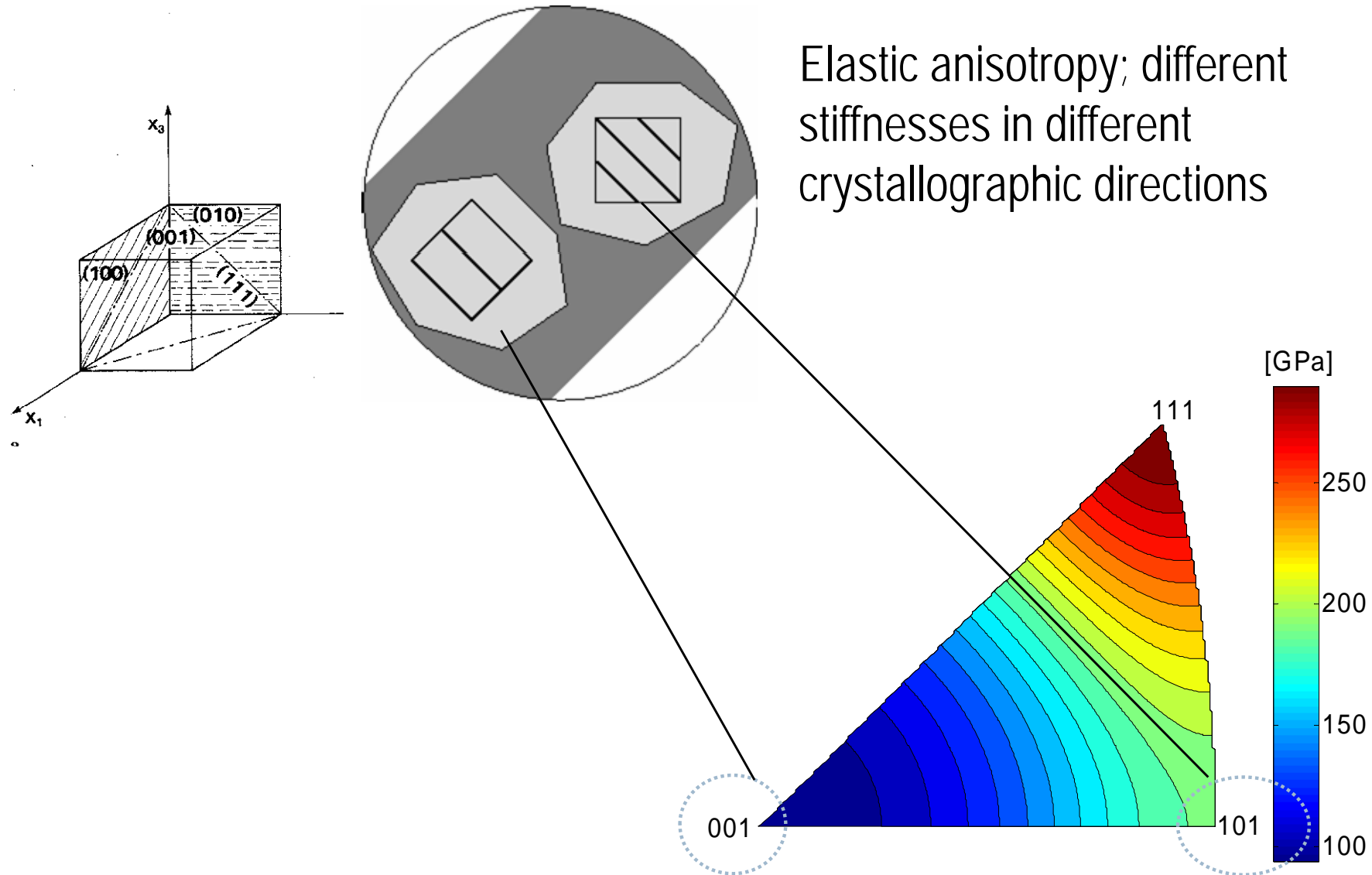
e.g. GSAS:

“RSTR” – isotropic strain parameter,

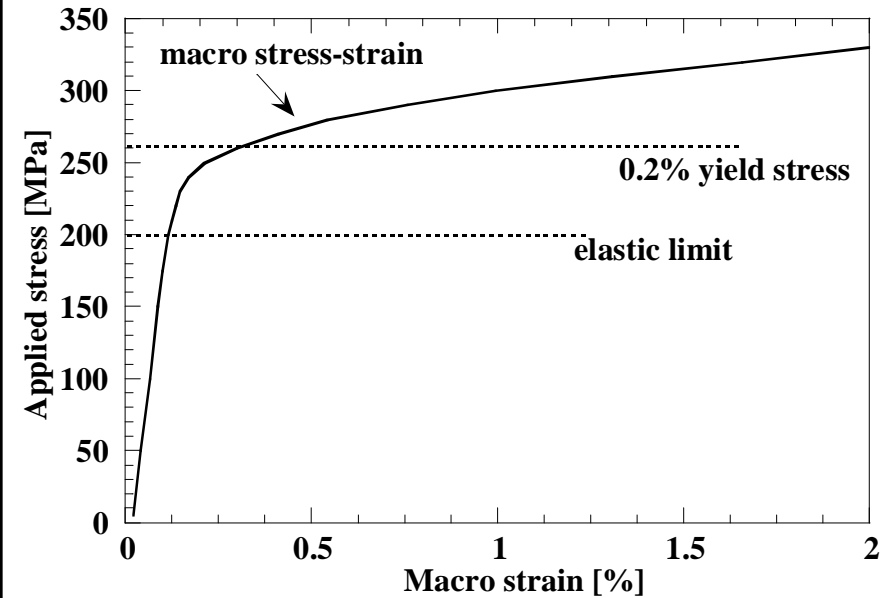
“RSTA” – assumes cosine variation with angle from c-axis



Elastic anisotropy

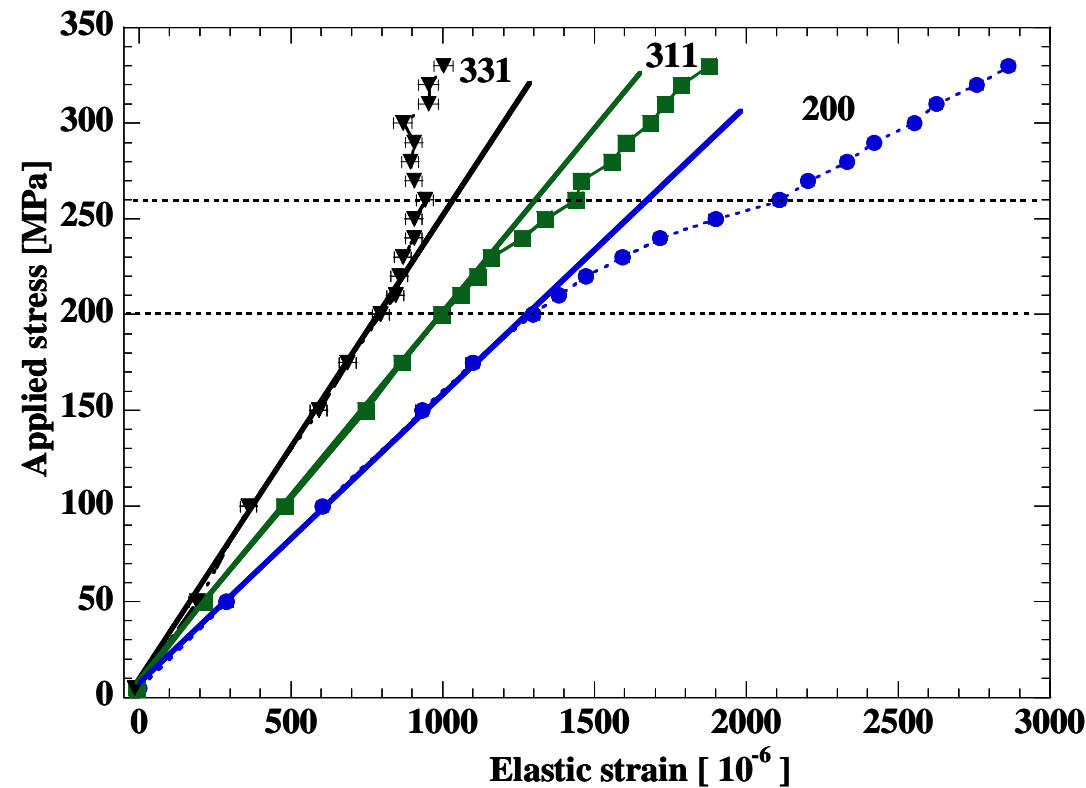


Plastic anisotropy

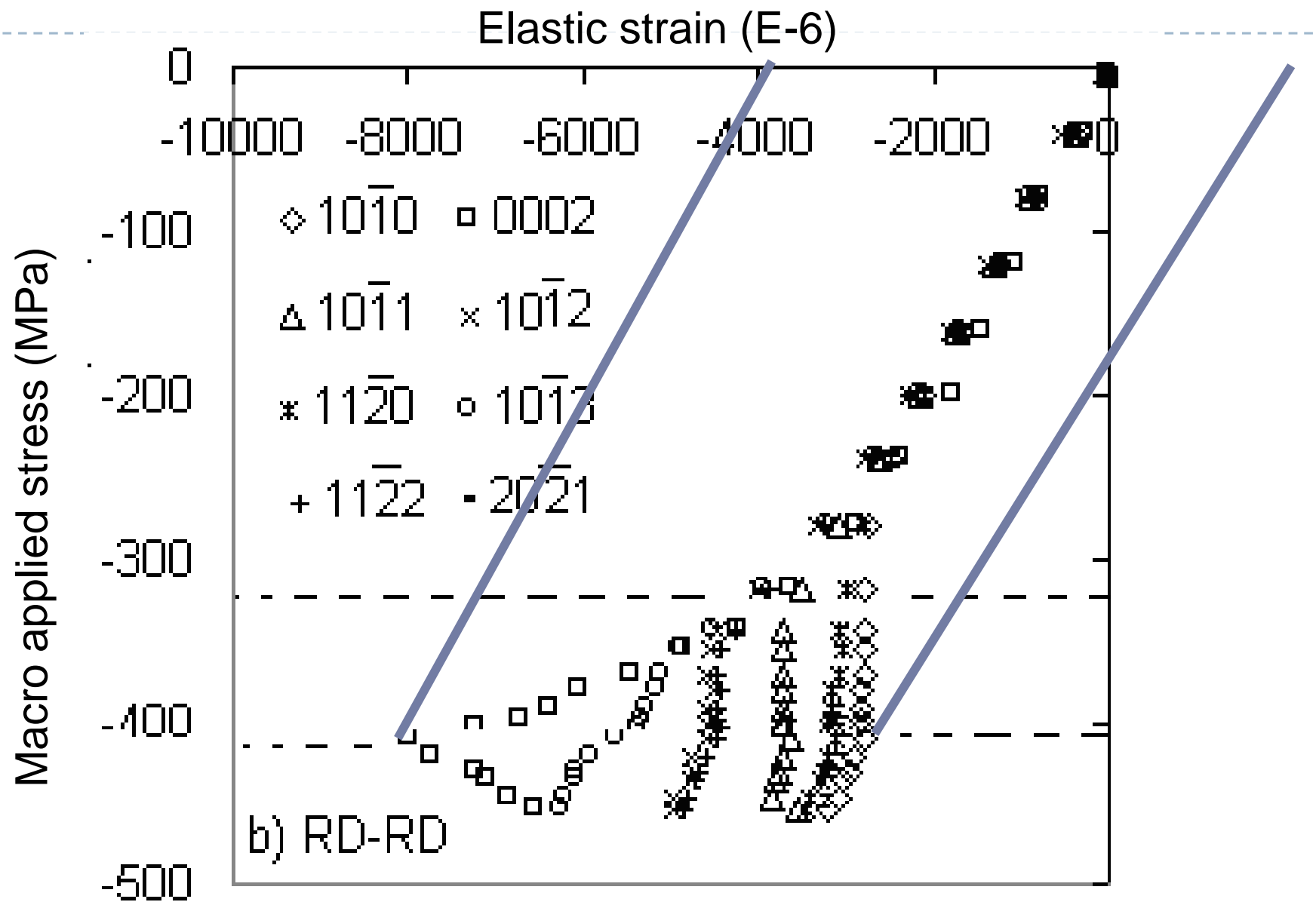


fcc structure
 $\{111\}\langle 110\rangle$ slip

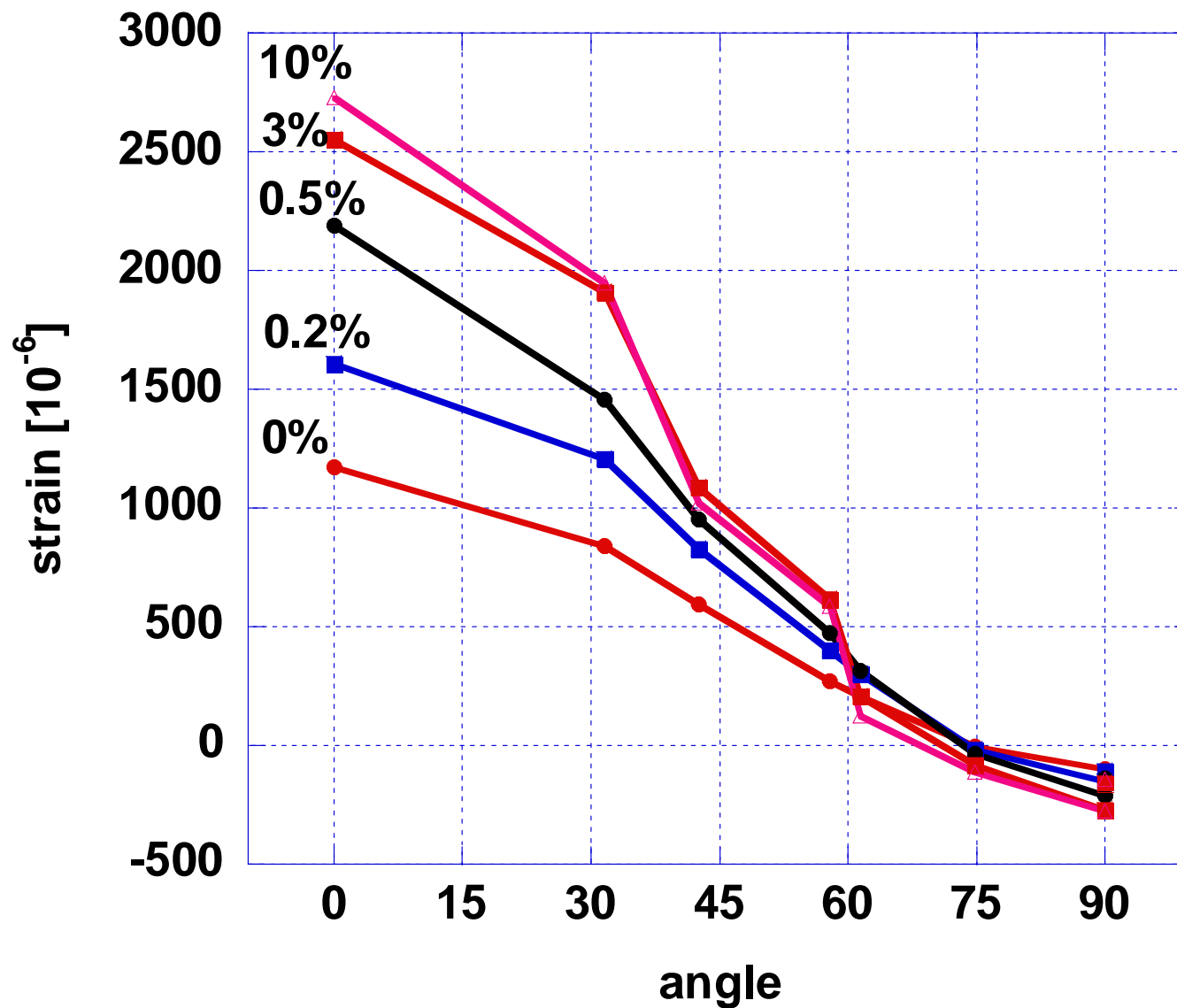
Plastic anisotropy;
introduces non-linearity



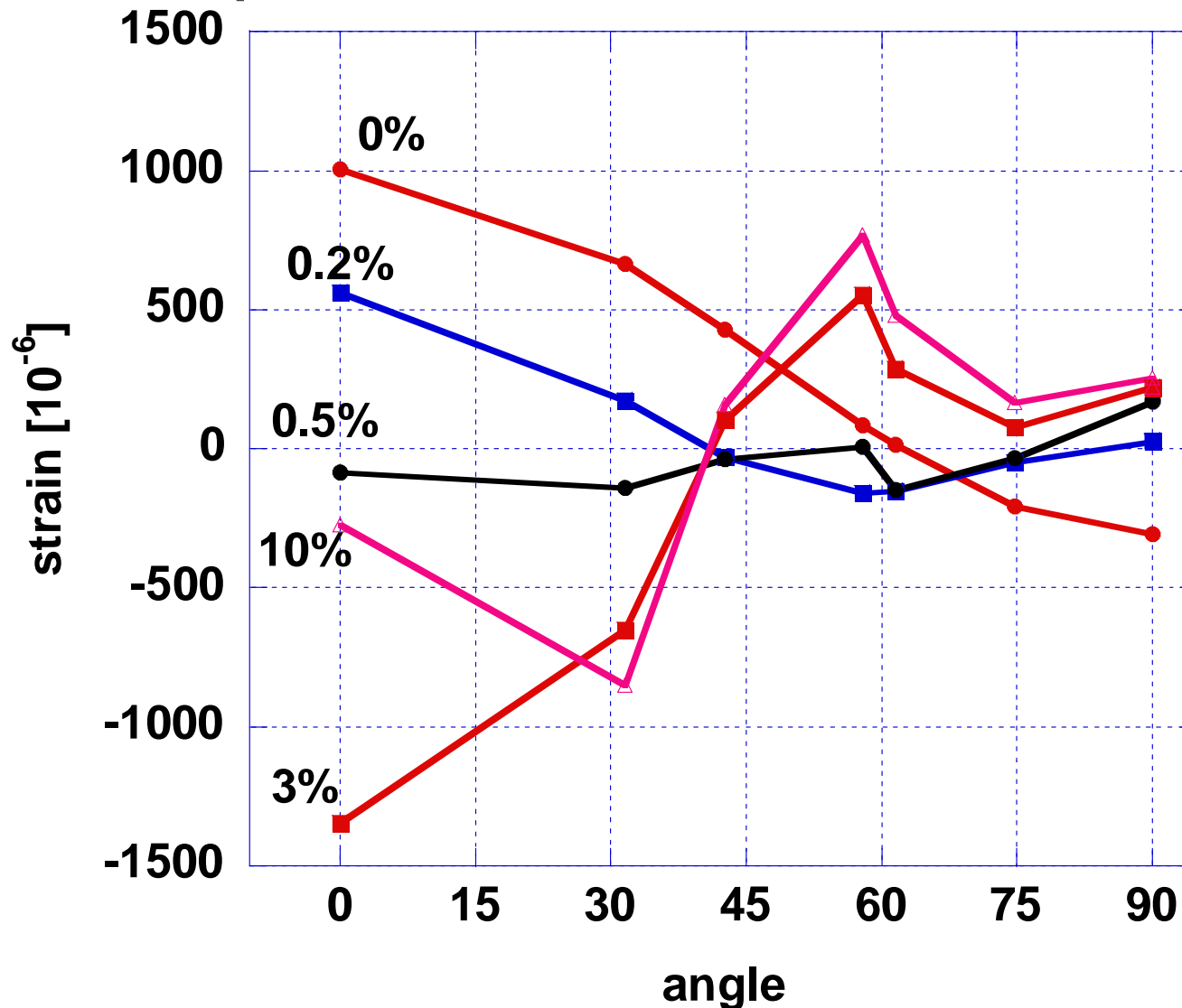
Examples of intergranular strains in Zr



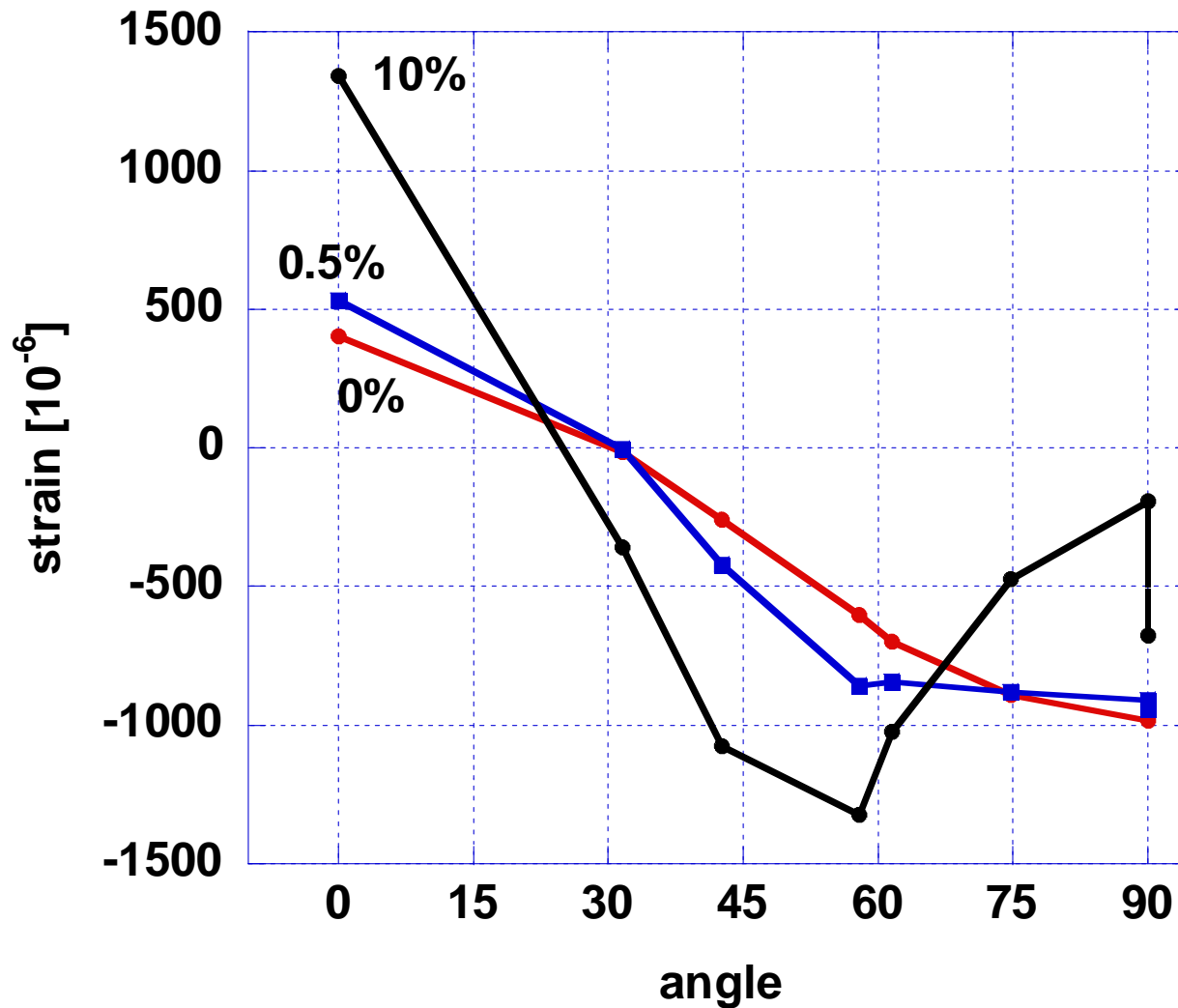
Deviation from ideal relation due to: plasticity induced stresses



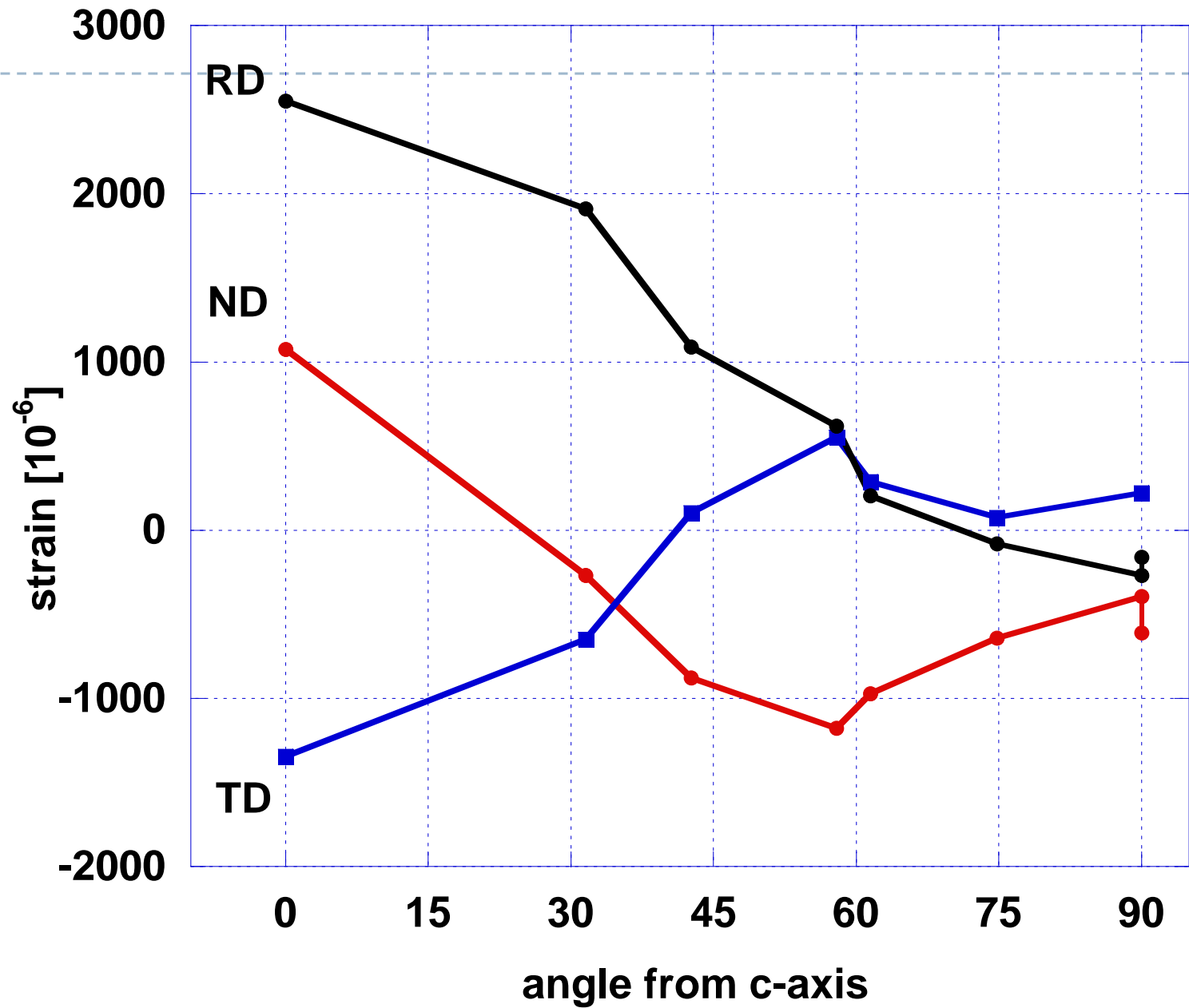
Deviation from ideal relation due to: plasticity induced stresses – response depends on texture



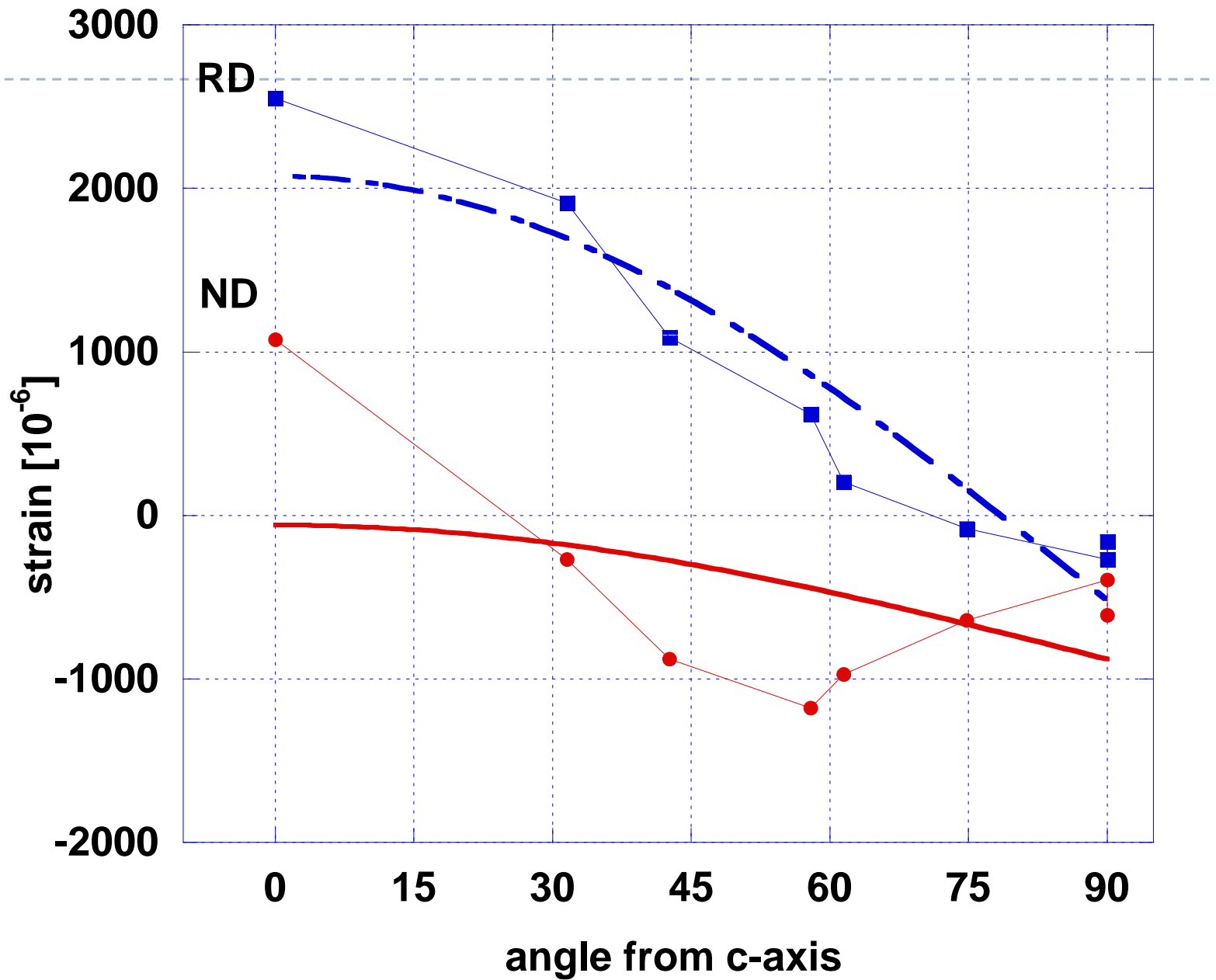
Deviation from ideal relation due to: plasticity induced stresses – response depends on texture



After 3% strain



After 3% strain



How to handle in a Rietveld refinement?

Most extreme peaks, typically strain of ~ 0.001

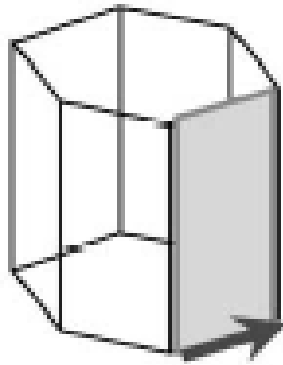
Response differs too far from relation to allow accurate peak width and/or peak intensity fit while maintaining peak positions

e.g. pseudo-anisotropic broadening

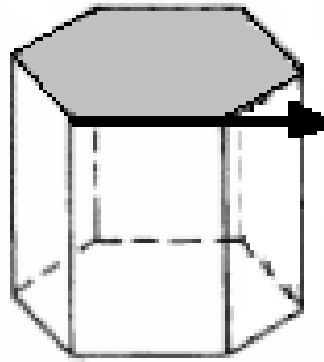
Solution

- 1) Allow peak positions to vary freely?
- 2) Incorporate models of plasticity?

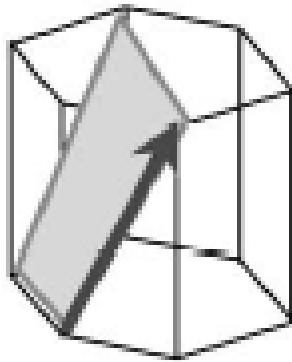
Deformation modes observed in single crystal hcp Zr



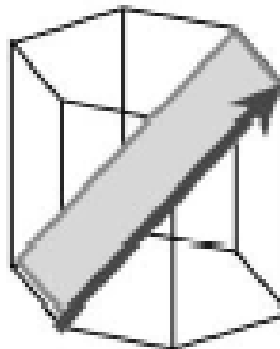
$(1010) \langle 1120 \rangle$
prismatic slip



$(0001) \langle 1120 \rangle$
basal slip



$(1011) \langle 1123 \rangle$
pyramidal slip



$(1012) \langle 1011 \rangle$
tensile twin

Understanding strains – models of polycrystalline plasticity

Single crystal properties

+

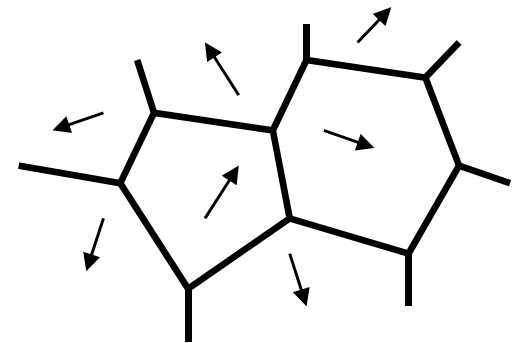
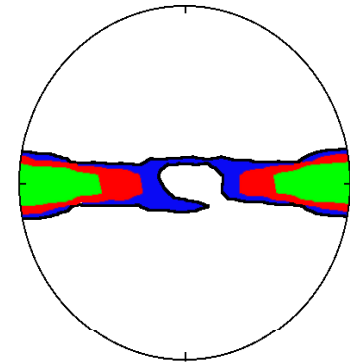
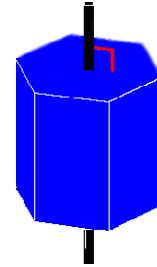
Crystallographic texture (ODF)

+

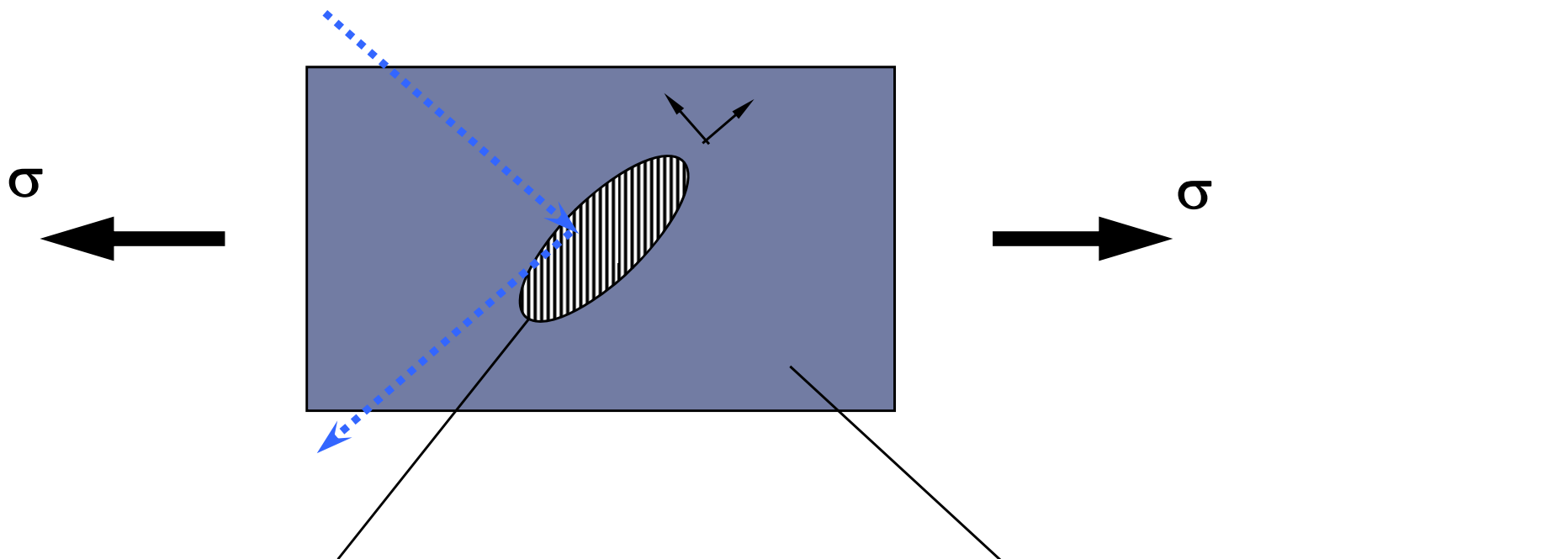
Crystal interactions w' matrix



Macroscopic & polycrystal properties



e.g. self-consistent elasto-plastic



Single crystallite

- elastic and plastic anisotropy
- relate crystallite strain / stress state to macro applied strain / stress.

Matrix

- homogenous average of all the crystallites
- iterate to solution

Model: self-consistent elasto-plastic

Keep track of each grain, its stress state and its plastic strain.

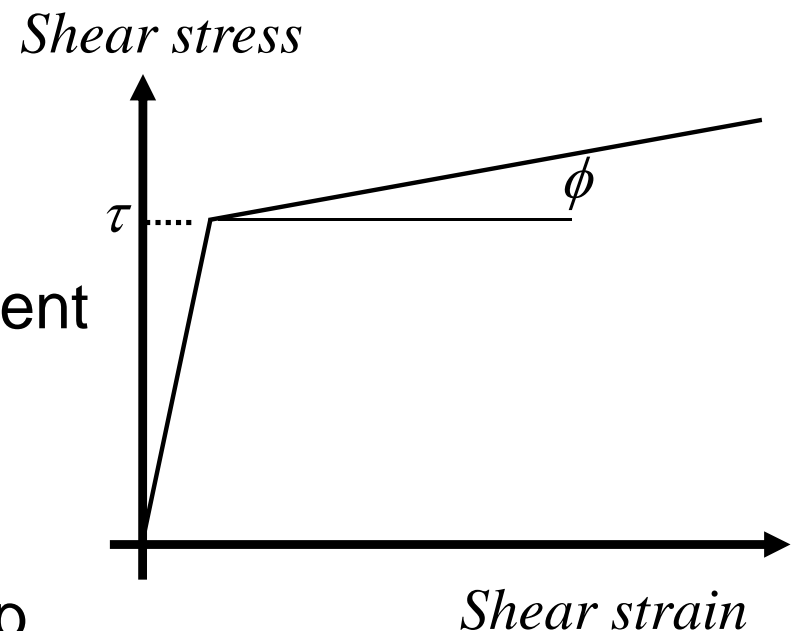
Fixed parameters:

- Single crystal elastic constants
- Texture (\Rightarrow grain population)
- Plastic slip directions / planes

Fitted parameters, to get best agreement with experiment

- Critical resolved shear stress, τ
- Hardening gradient ϕ
i.e. only 2 fitting parameters for a slip system

(Perhaps more complex plastic law)



Use of zirconium

Reactors worldwide

– Zircaloy Fuel Cladding;

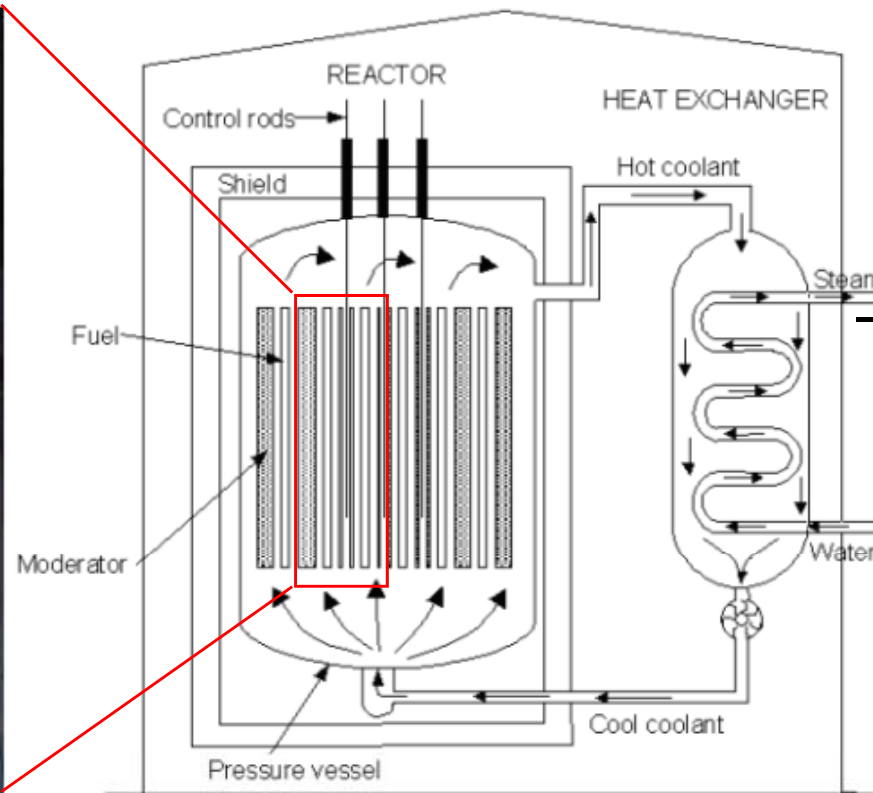
- Pressurized or unpressurized H₂O coolant
- Temperatures range from ambient to >300°C

– PHWR;

- Zr-2.5Nb* pressure tube contains coolant
- Zircaloy-2 'calandria' tube, separates hot pressure tube from moderator

– Research reactors;

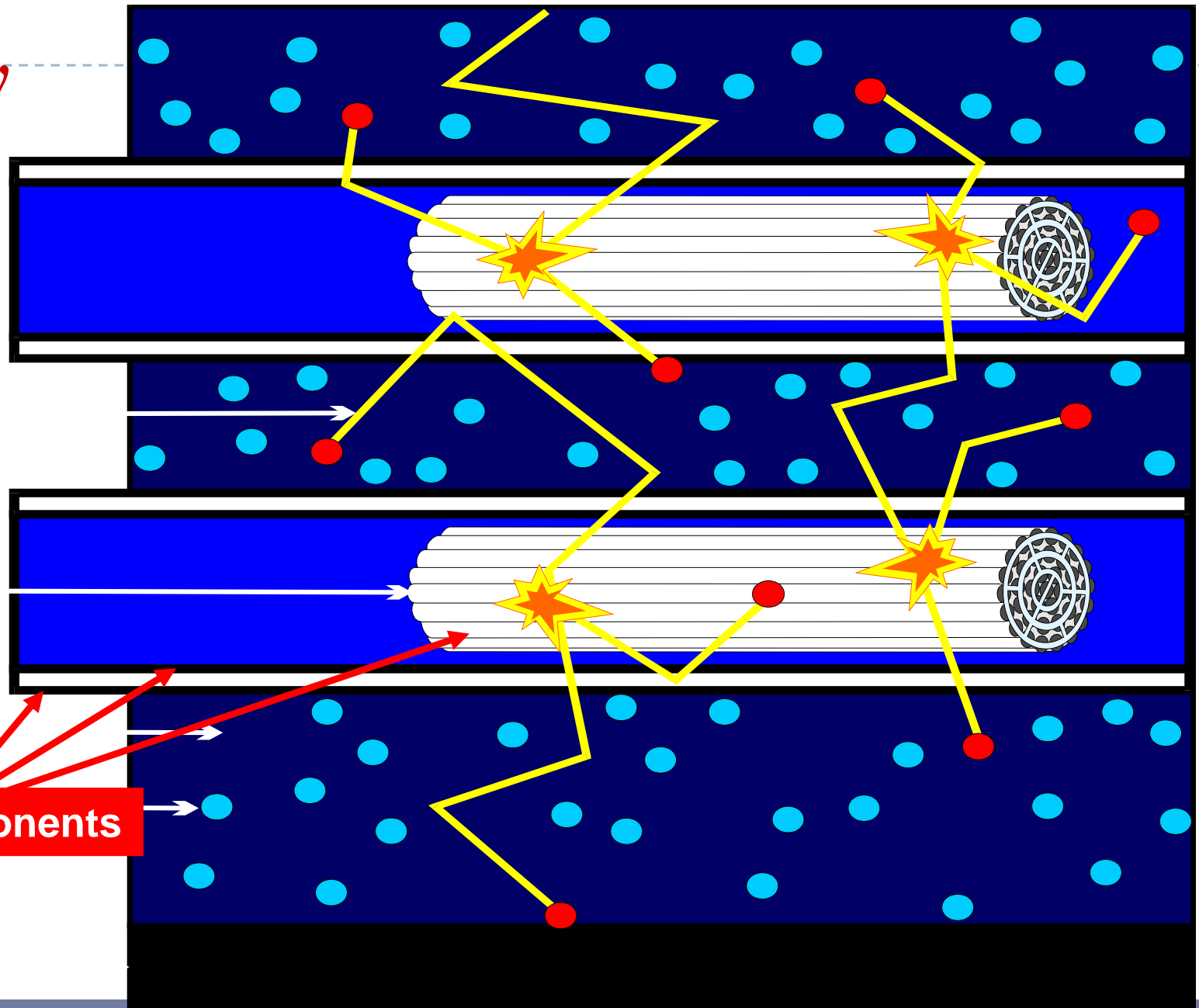
- Reflector vessel walls



Metallurgy somewhat analogous to titanium

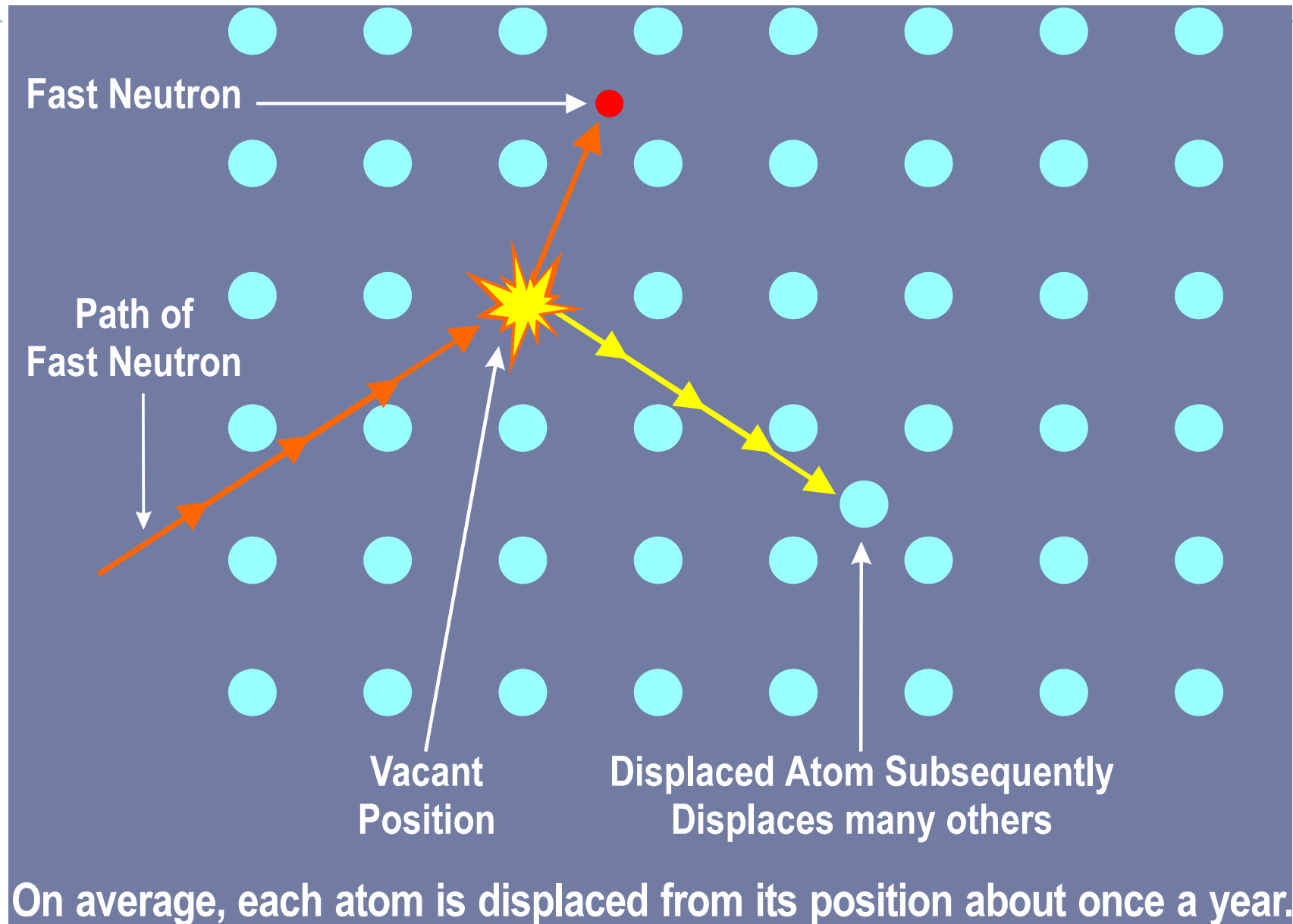
* 2 phase alloy, c.f. Ti-6Al-4V

Neutron Economy



Zr alloy components

Effect of neutron irradiation



Residual structure after decay of thermal spike

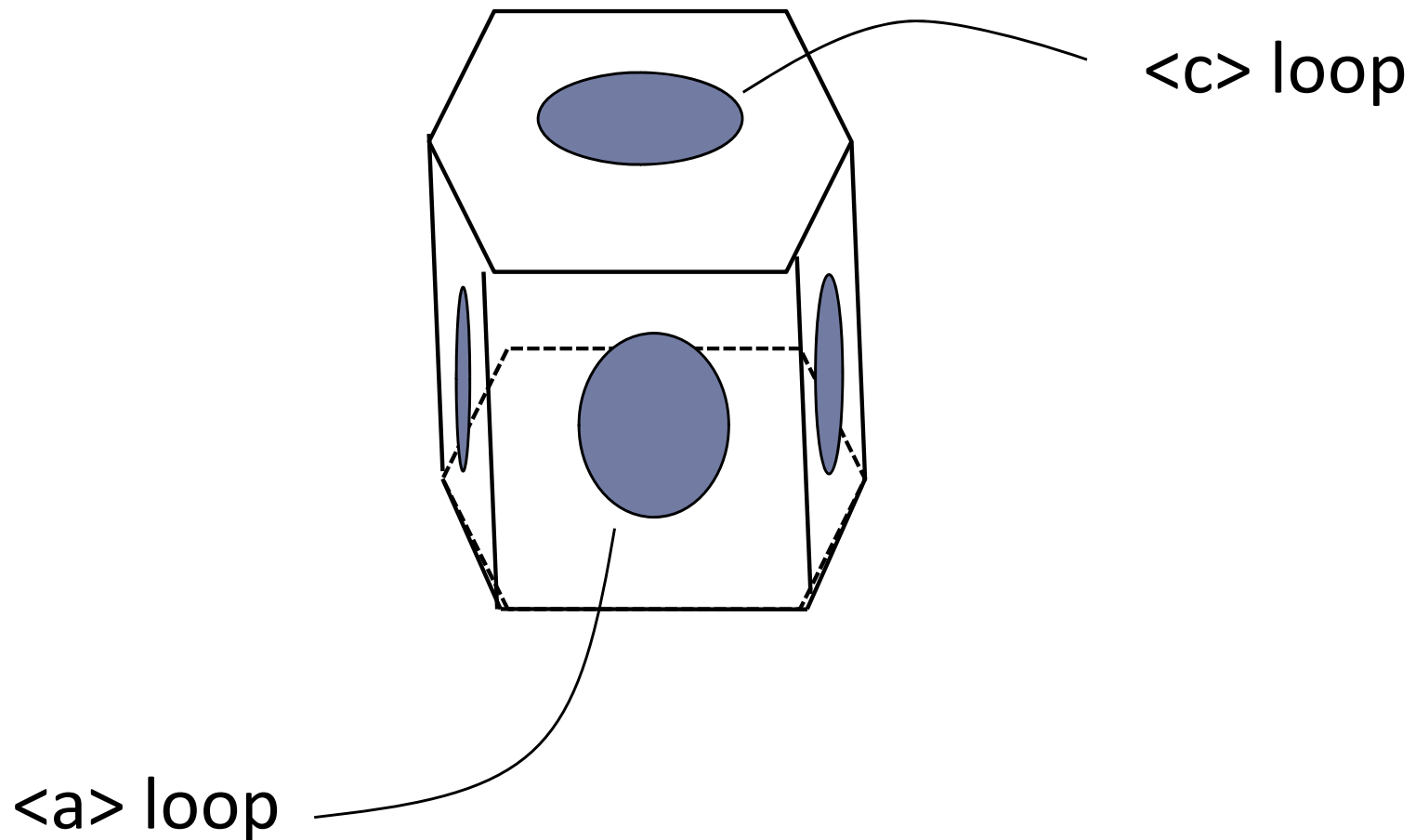
- ▶ Vacancy clusters
- ▶ Interstitial clusters
- ▶ Individual vacancies (freely migrating)
- ▶ Individual SIAs (freely migrating)

Subsequent Evolution

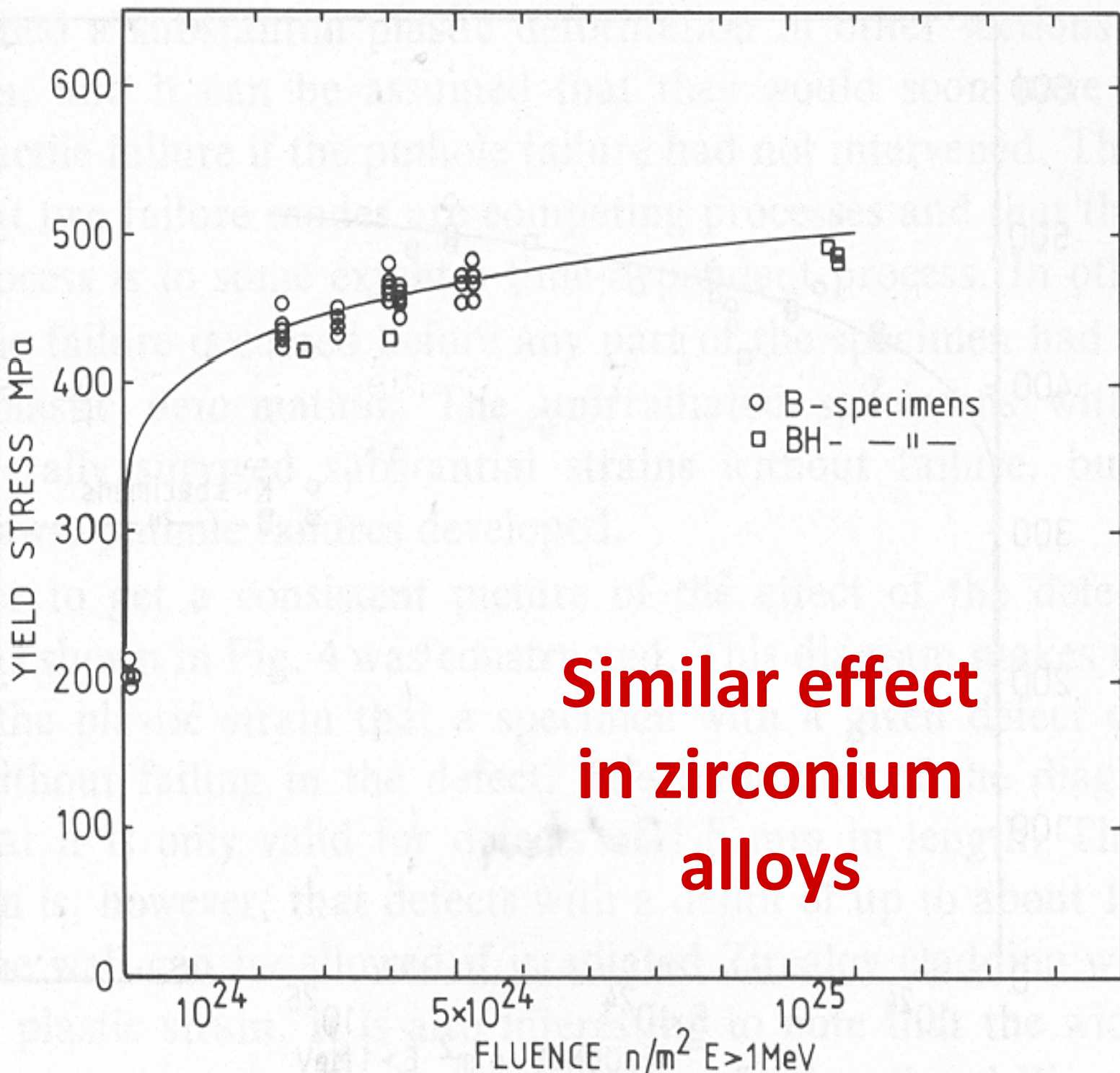
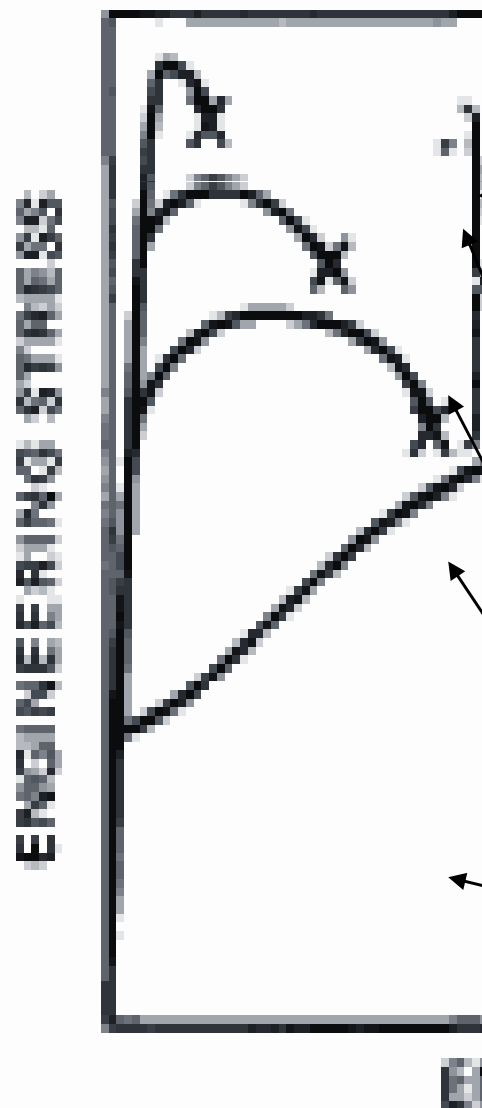
- Vacancies and interstitials migrate through the lattice and the microstructure evolves
- Usually a dense dislocation “loop” structure generated over first few dpa

Dislocation loops

Differences in migration rates / anisotropy lead to
different densities of $\langle a \rangle$ and $\langle c \rangle$ loop dislocations

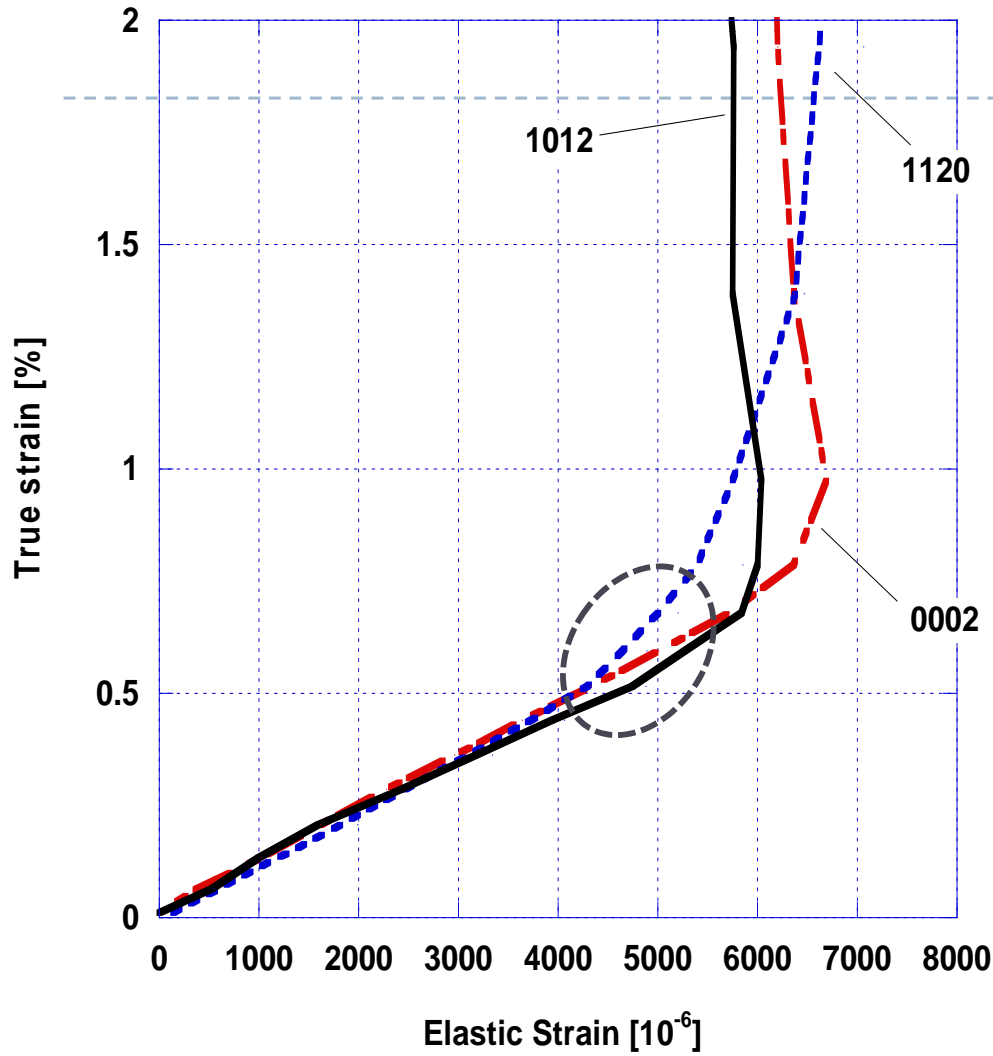


Hardening



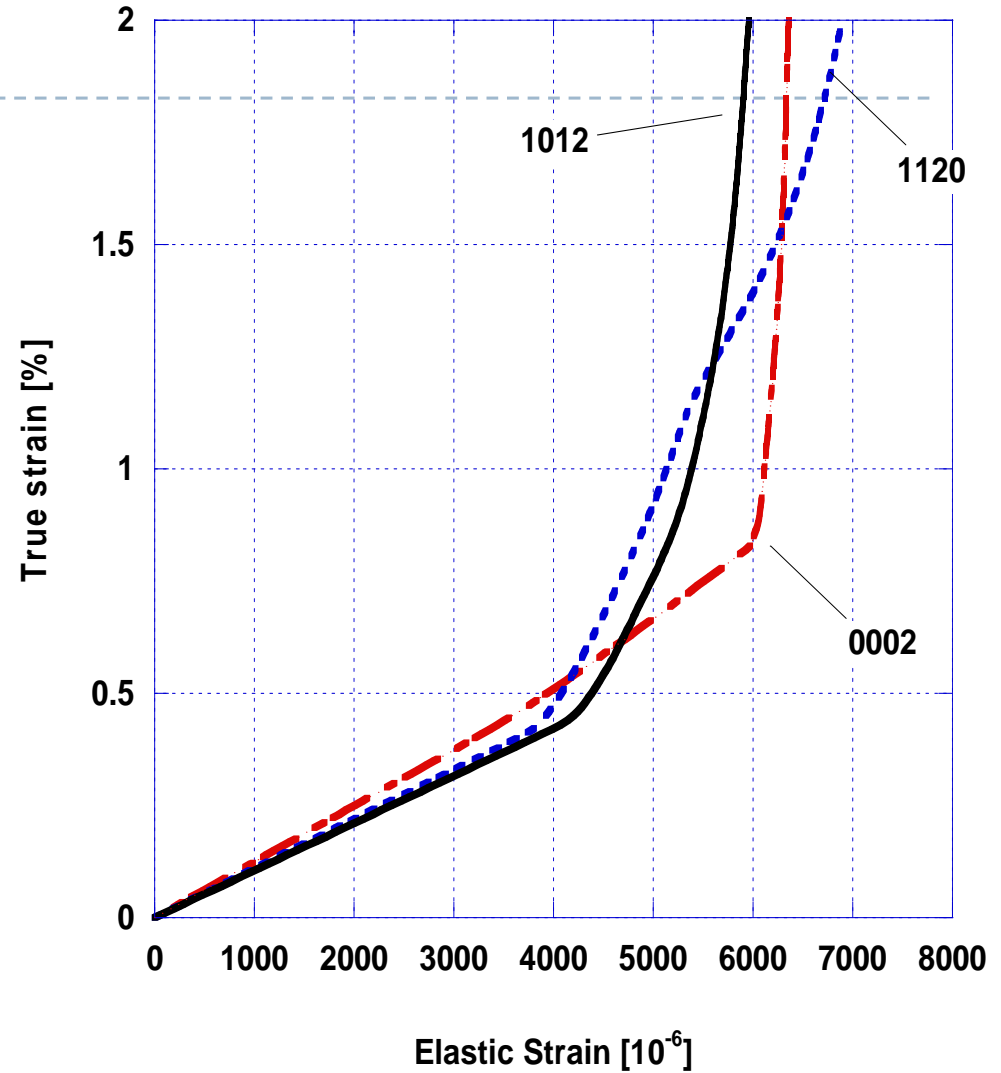
**Similar effect
in zirconium
alloys**

Unirradiated Experiment, Load Parallel to Hoop



Experiment

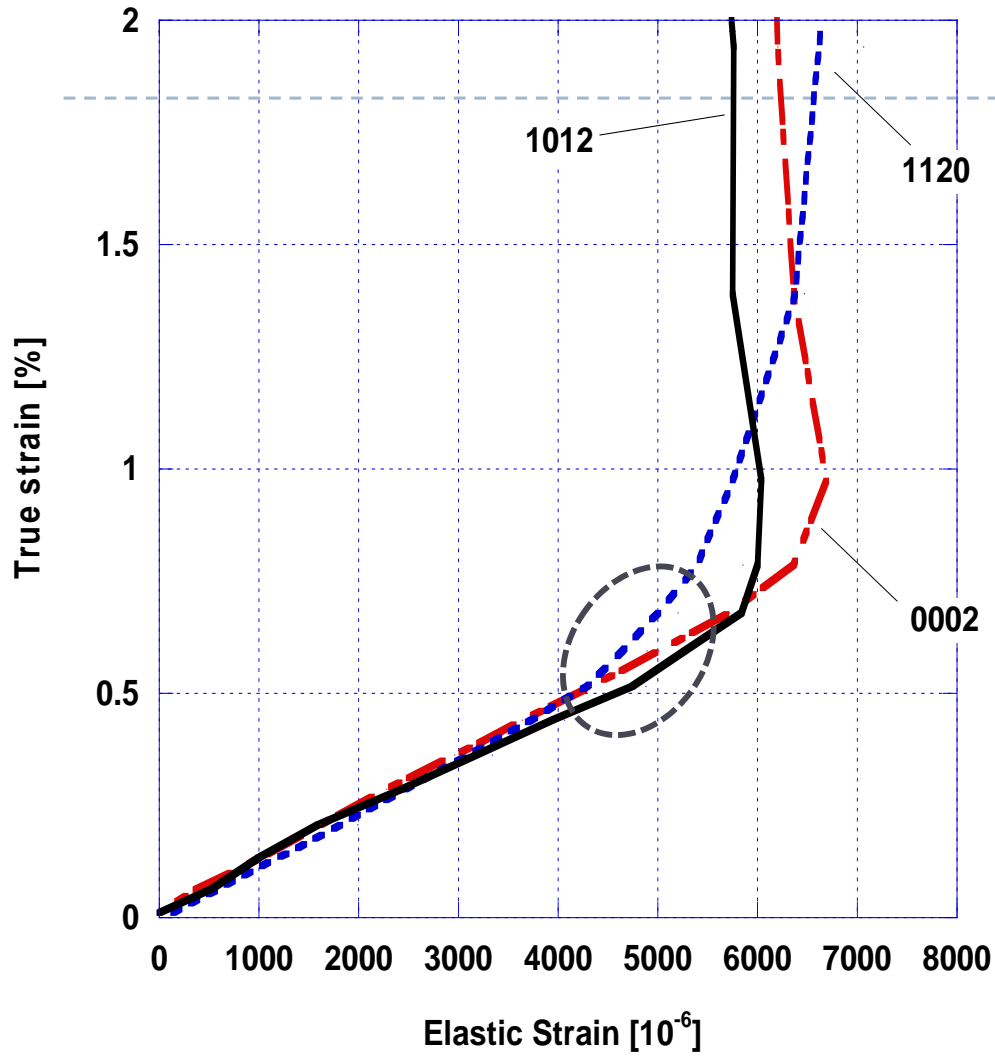
Unirradiated Model, Load Parallel to Hoop



Model

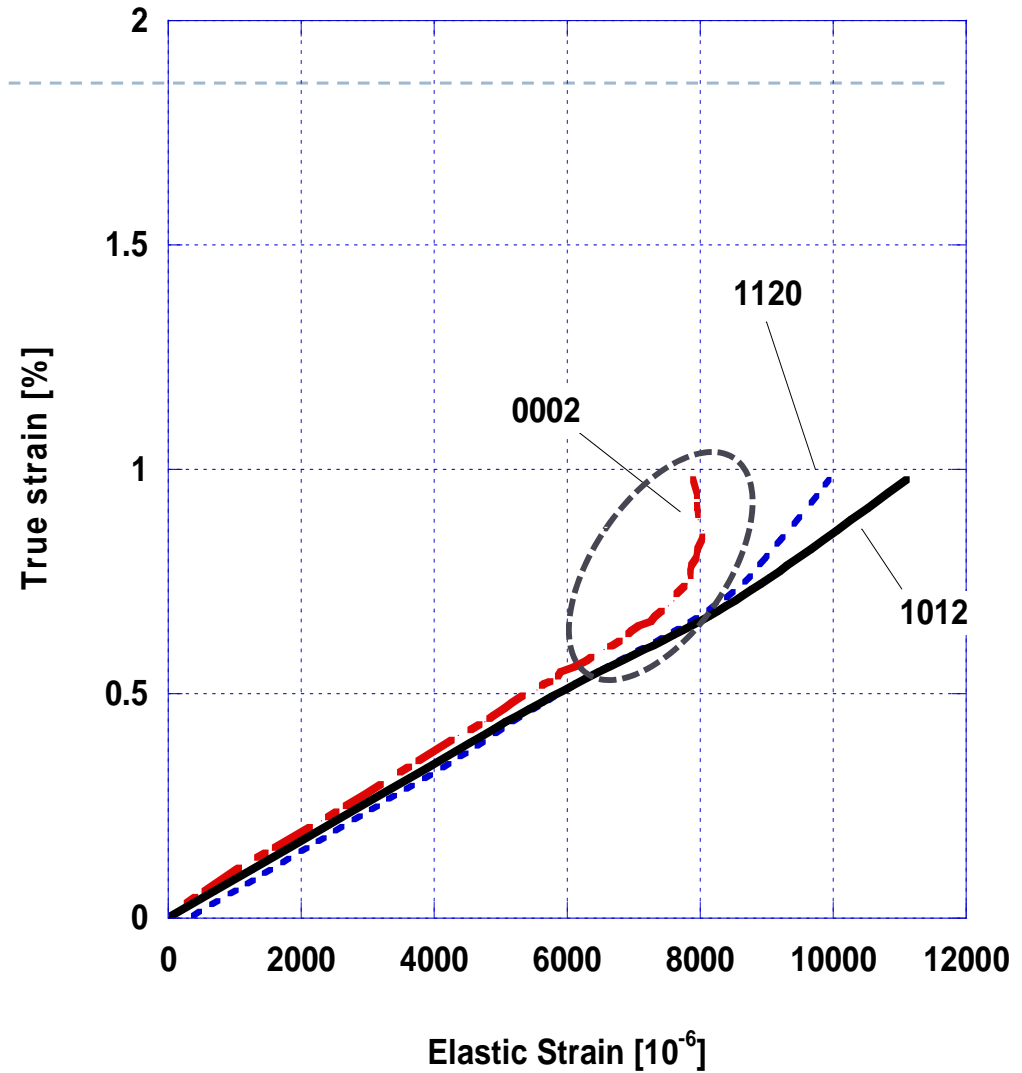
Very low hardening – plot vs macroscopic strain

Unirradiated Experiment, Load Parallel to Hoop



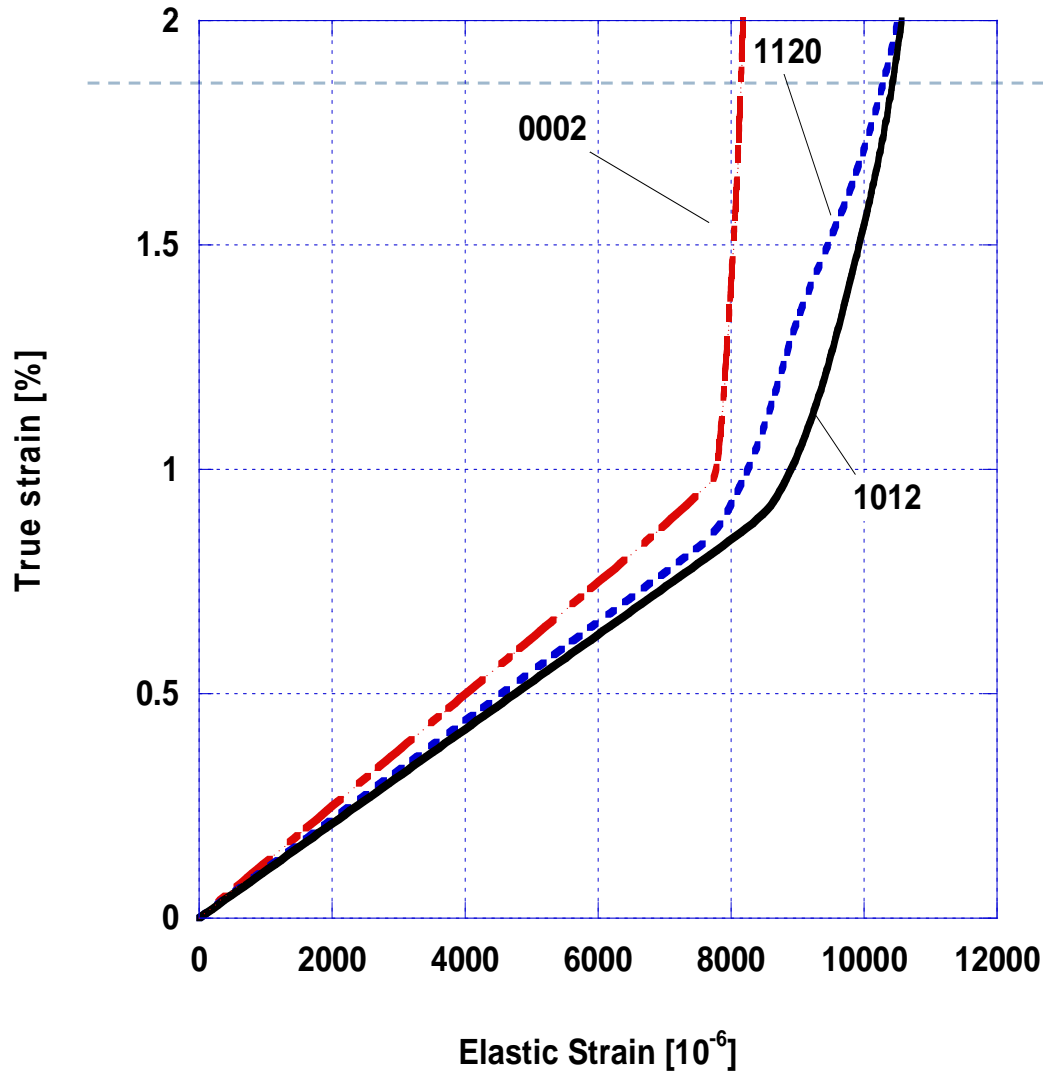
Unirrad Experiment

Irradiated Experiment, Load Parallel to Hoop



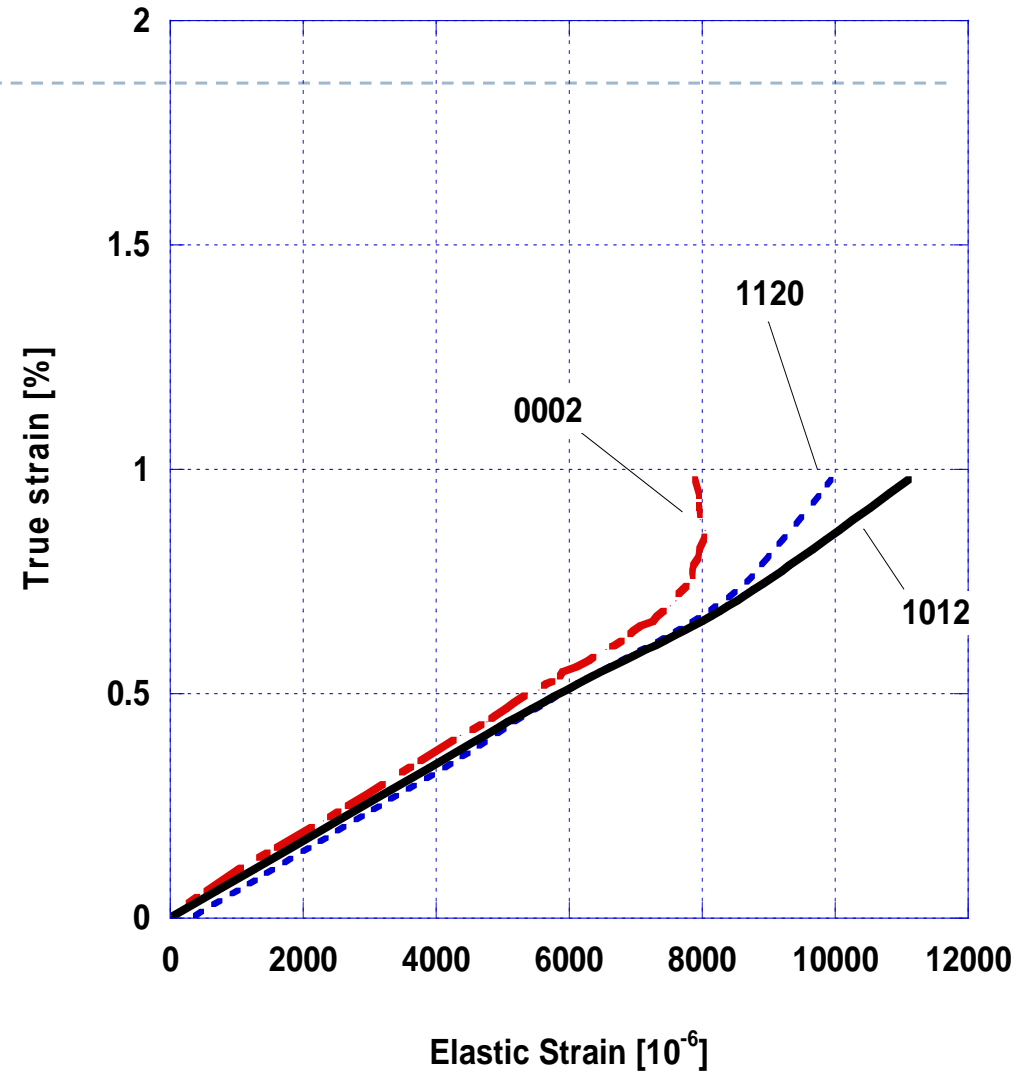
Irrad experiment

Irradiated Model 1, Load Parallel to Hoop



Irrad Model

Irradiated Experiment, Load Parallel to Hoop

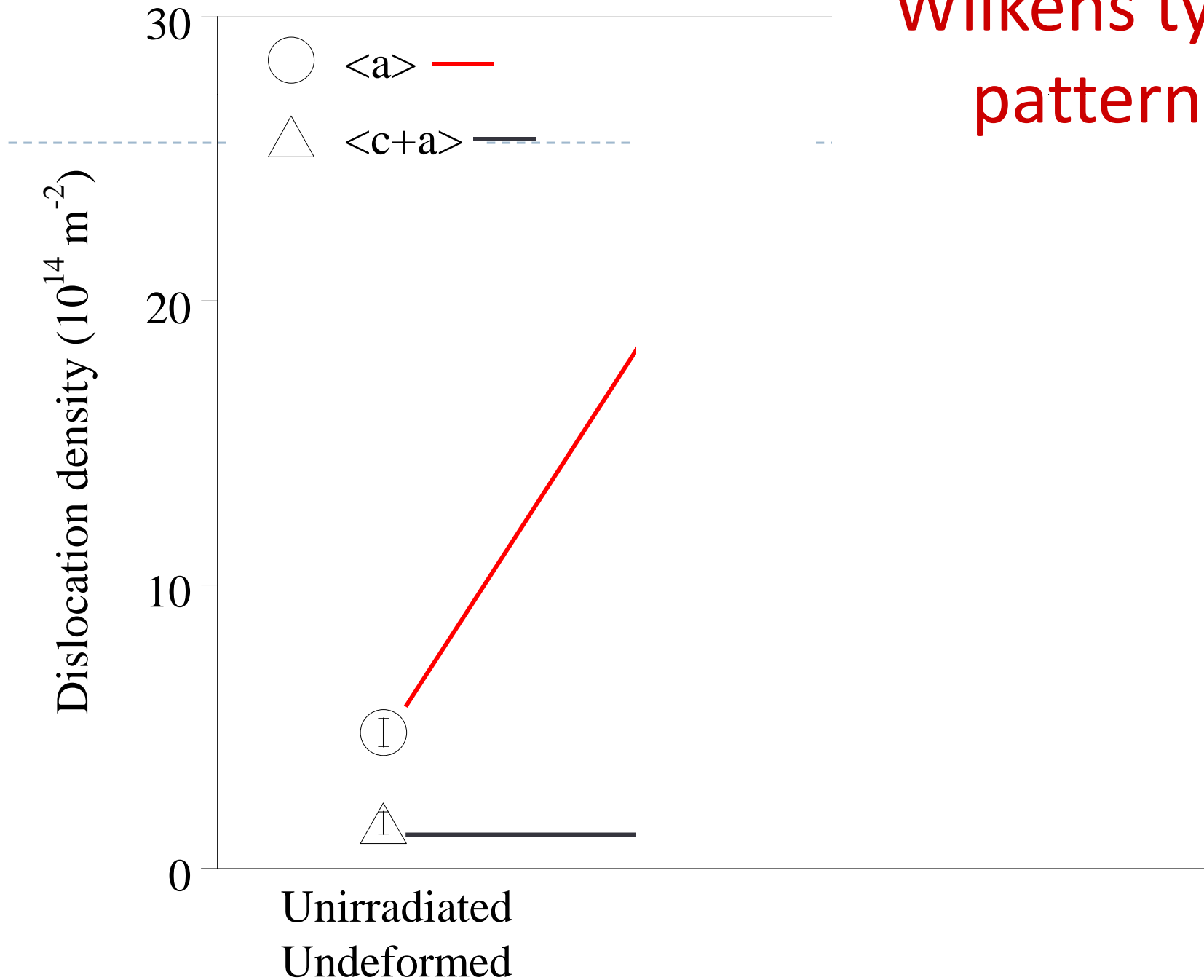


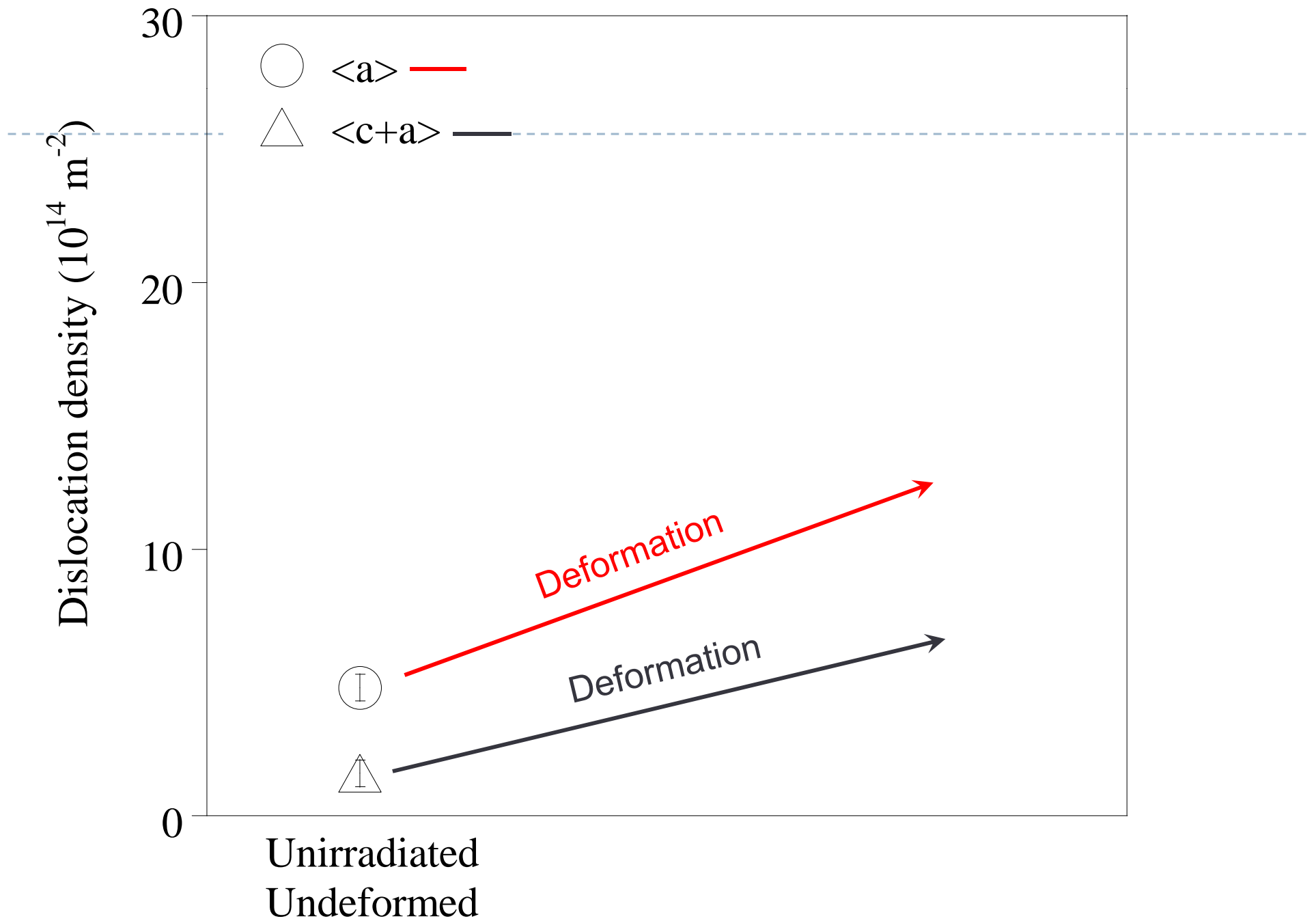
Irrad experiment

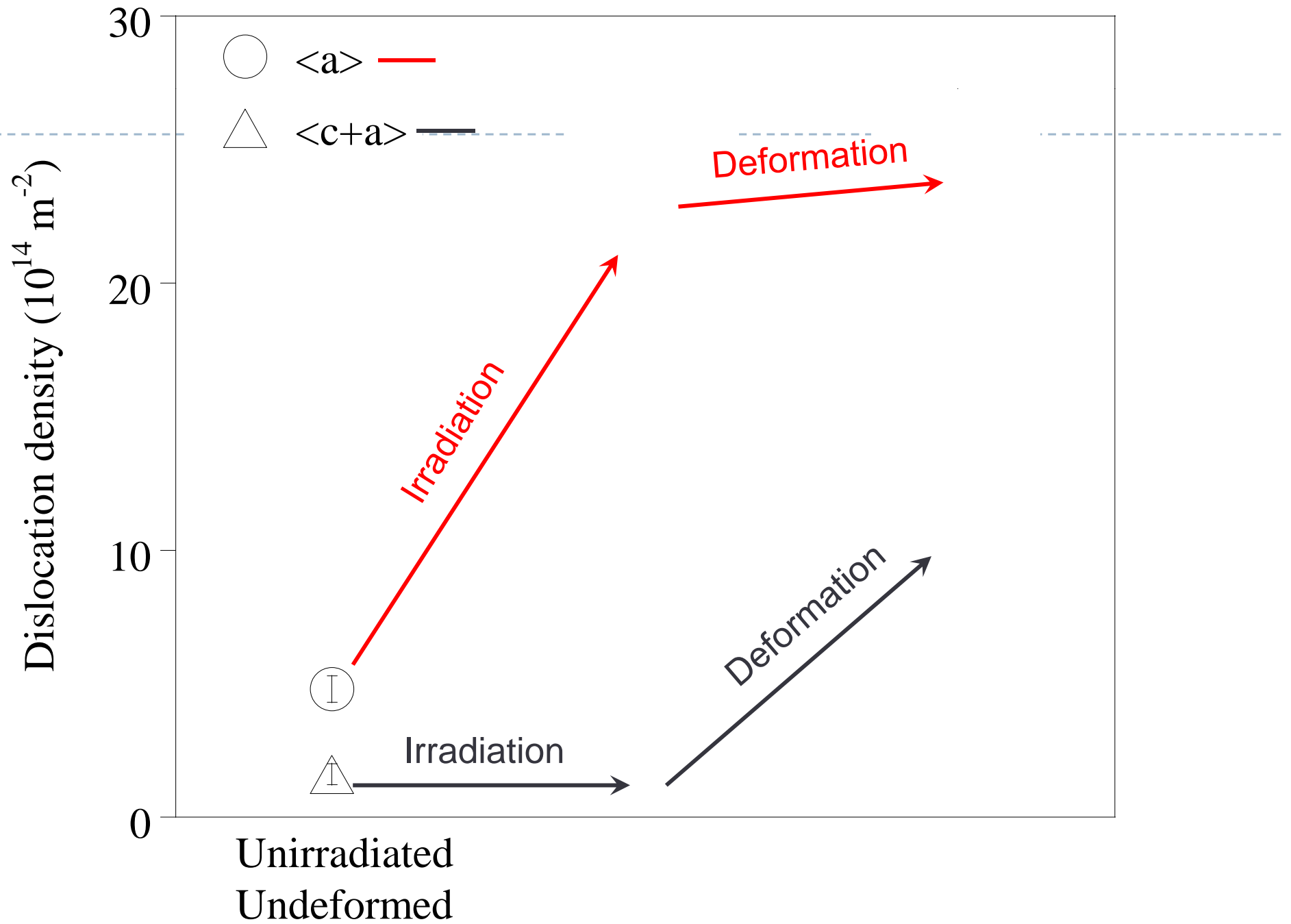
Impact of irradiation on model (fit) CRSS

Mode	Unirradiated [MPa]	Irradiated [MPa]	Increase [MPa]
Prism <a>	160	370	210 (x2.3)
Basal <a>	170	370	200 (x2.2)
Tensile twin	450	<i>n/a</i>	<i>n/a</i>
<c+a>	460	500	0 (x1.1)
<i>Macroscopic (experimental 0.2% yield)</i>	<i>530 / 740</i>	<i>820 / 950</i>	<i>(x1.5 / x1.3)</i>

Wilkens type full pattern LPA







Conclusion

- ▶ Internal stresses change location of peaks enough that we must account for them in Rietveld refinement
- ▶ Can study these changes to determine crystal
 - ▶ Irradiation changed response (load sharing) of differently (crystallographically) oriented grains.
 - ▶ Effect of irradiation on critical resolved shear stress in extruded Zr_{2.5}Nb was determined
- ▶ Can be correlated with dislocation populations measured by peak width analysis (typically ex situ)

Acknowledgements

- ▶ Data collected at
 - ▶ ISIS (Ed Oliver)
 - ▶ LANSCE (Bjorn Clausen)
- ▶ Fei Long, Feng Xu – Queen's
- ▶ Levente Balogh - LANSCE

- ▶ Work funded by UNENE / NSERC Industrial Research Chair in Nuclear Materials