

IONIZING RADIATION SAFETY – RADIOACTIVE MATERIALS AT NIST- BOULDER

NIST S 7201.02

Approval Date: 02/10/2017

Effective Date:¹ 02/10/2017

Table of Contents

10	1. PURPOSE	2
11	2. BACKGROUND	2
12	3. APPLICABILITY	3
13	4. REFERENCES	3
14	5. APPLICABLE NIST OCCUPATIONAL SAFETY AND HEALTH DIRECTIVES	4
15	6. DEFINITIONS	4
16	7. ACRONYMS	11
17	8. REQUIREMENTS	12
18	a. Radiation Safety Training	13
19	b. Radiation Monitoring Instruments	17
20	c. Radiological Surveys	18
21	d. Material Receipt, Accountability, & Security	20
22	e. Safe Use of Radionuclides and Emergency Procedures	21
23	f. Occupational Dose	22
24	g. Transportation	23
25	h. Waste Management	23
26	i. Program Review	23
27	j. Notifications and Reports	24
28	k. Recordkeeping	24
29	9. RESPONSIBILITIES	25
30	10. AUTHORITIES & ACCOUNTABILITIES	28
31	11. DIRECTIVE OWNER	29
32	12. APPENDICES	29
33	Appendix A. Revision History	30

¹ For revision history, see Appendix A.

35 **1. PURPOSE**

36 a. This suborder establishes the requirements of an operational radiation safety program
37 necessary for the implementation of NIST Order 7201 as it applies to the acquisition, use,
38 and disposition of radioactive material (RAM) at NIST Boulder. The main goal of the
39 program is to permit the beneficial use of radioactive materials for research and development
40 purposes while ensuring doses are maintained as low as is reasonably achievable (ALARA).

41
42 b. This suborder establishes:

- 43
44 (1) Program elements;
45
46 (2) Roles of individuals and their corresponding responsibilities;
47
48 (3) Authorities necessary for fulfilling responsibilities; and
49
50 (4) A standard of accountability.

51
52 c. This suborder describes the “what’s required” and “who’s responsible” for the program. The
53 implementation (the “how to”) of this program is accomplished through various documents
54 such as a radiation safety manual, procedures, instructions, guideline, *etc.*

55
56

57 **2. BACKGROUND**

58 a. As a federal organization, NIST is subject to the authority of the Nuclear Regulatory
59 Commission (NRC) regulations regarding radioactive materials written in Title 10, Code of
60 Federal Regulations (CFR).

61
62 b. The acquisition, use, and disposition of licensed radioactive material is authorized by the
63 issuance of a specific license or granted through a general license. NIST Boulder is eligible
64 for both types of licenses. Licensed material is used in accordance with a specific license;
65 generally-licensed materials and other materials are used in accordance with applicable
66 regulations.

67
68 c. NIST may revise radiation safety program procedures without submitting an application for
69 license amendment provided that:

- 70
71 (1) The changes are approved by the Radiation Safety Officer (RSO) in writing;

72

73 (2) The appropriate licensee staff members are provided training in the revised procedure(s)
74 prior to implementation;

75
76 (3) The changes are in compliance with the NRC regulations and the license; and
77

78 (4) The changes do not degrade the effectiveness of the program. [LC20A Item 10]
79

80 (a) Program requirements imposed on the use of specifically-licensed material by NRC
81 Materials License 05-03166-06 are emphasized by the format “[License Condition #
82 and in some cases specific sections]” (e.g., LC20A Item 10 means License Condition
83 20A. Application dated April 28, 2015 (ML15156B315) Item 10 Radiation Safety
84 Program) since the condition may be more restrictive than regulation.
85

86 87 **3. APPLICABILITY**

88 This suborder applies to activities conducted under specific licenses applicable to NIST Boulder
89 or involving generally-licensed or non-specially-licensed RAM regulated by the NRC.
90

91 92 **4. REFERENCES**

93 a. Code of Federal Regulations, Energy Title 10, [Chapter 1](#), various parts.
94

95 b. Code of Federal Regulations, Protection of Environment [Title 40](#), as applicable.
96

97 c. Code of Federal Regulations, Transportation [Title 49](#), as applicable.
98

99 d. NRC Materials License(s), current.
100

101 e. U.S. NRC Consolidated Guidance About Materials Licenses ([NUREG-1556](#)).
102

103 f. [NIST O 7201.00](#), Ionizing Radiation Safety – Radioactive Material and Ionizing-Radiation-
104 Producing Machines.
105

106 g. U.S. NRC [Regulatory Guide 8.13](#) Instruction Concerning Prenatal Exposure.
107

108 h. U.S. NRC [Regulatory Guide 8.29](#) Instruction Concerning Risks from Occupational Radiation
109 Exposure.
110

111 i. ANSI N323-1978 Radiation Protection Instrumentation Test and Calibration.

112 j. National Council on Radiation Protection and Measurements (NCRP) Report No. 127
113 Operational Radiation Safety Program.

114
115

116 5. APPLICABLE NIST OCCUPATIONAL SAFETY AND HEALTH DIRECTIVES

117 a. NIST S 7101.20: *Work and Worker Authorization Based on Hazard Reviews.*

118

119 b. NIST S 7101.23: *Safety Education and Training Program.*

120

121 c. NIST S 7101.24: *Incident Reporting and Investigation.*

122

123 d. NIST S 7101.04: *Safety and Health Requirements for Minors.*

124

125

126 6. DEFINITIONS

127 a. **ALARA** (acronym for "as low as is reasonably achievable") - Making every reasonable
128 effort to maintain exposures to radiation as far below the dose limits in this part as is
129 practical consistent with the purpose for which the licensed activity is undertaken, taking into
130 account the state of technology, the economics of improvements in relation to state of
131 technology, the economics of improvements in relation to benefits to the public health and
132 safety, and other societal and socioeconomic considerations, and in relation to utilization of
133 nuclear energy and licensed materials in the public interest.

134

135 b. **Ancillary Personnel** – Individuals who are not actively fulfilling the role of an AU or SU
136 but whose assigned duties involve potential exposure to radiation and/or radioactive material.

137

138 c. **Authorized User** (sometimes referred to as the Principal Investigator (PI)) – The NRC-
139 approved individual who is assigned primary responsibility for the safe use and handling of
140 specifically-licensed RAM they have been approved to possess.

141

142 d. **Becquerel (Bq)** – The SI unit of radioactivity, corresponding to one disintegration per
143 second.

144

145 e. **Byproduct material** –

146 (1) Any radioactive material (except special nuclear material) yielded in, or made radioactive
147 by, exposure to the radiation incident to the process of producing or using special nuclear
148 material;

149

- 150 (2) The tailings or wastes produced by the extraction or concentration of uranium or thorium
151 from ore processed primarily for its source material content, including discrete surface
152 wastes resulting from uranium solution extraction processes. Underground ore bodies
153 depleted by these solution extraction operations do not constitute "byproduct material"
154 within this definition;
155
- 156 (3) Any discrete source of radium-226 that is produced, extracted, or converted after
157 extraction, before, on, or after August 8, 2005, for use for a commercial, medical, or
158 research activity; or any material that:
159
- 160 (a) Has been made radioactive by use of a particle accelerator; and
161
- 162 (b) Is produced, extracted, or converted after extraction, before, on, or after August 8,
163 2005, for use for a commercial, medical, or research activity; and
164
- 165 (4) Any discrete source of naturally occurring radioactive material, other than source
166 material, that:
167
- 168 (a) The Commission, in consultation with the Administrator of the Environmental
169 Protection Agency, the Secretary of Energy, the Secretary of Homeland Security, and
170 the head of any other appropriate Federal agency, determines would pose a threat
171 similar to the threat posed by a discrete source of radium-226 to the public health and
172 safety or the common defense and security; and
173
- 174 (b) Before, on, or after August 8, 2005, is extracted or converted after extraction for use
175 in a commercial, medical, or research activity.
176
- 177 f. **Committed Dose Equivalent** ($H_{T,50}$) – The dose equivalent to organs or tissues of reference
178 (T) that will be received from an intake of radioactive material by an individual during the
179 50-year period following the intake.
180
- 181 g. **Controlled area** – An area, outside of a restricted area but inside the site boundary, access to
182 which can be limited by the licensee for any reason.
183
- 184 h. **Curie** (Ci) – A unit of radioactivity equal to 3.7×10^{10} disintegrations per second.
185
- 186 i. **Declared pregnant woman** – A woman who has voluntarily informed the licensee, in
187 writing, of her pregnancy and the estimated date of conception. The declaration remains in

188 effect until the declared pregnant woman withdraws the declaration in writing or is no longer
189 pregnant.

190

191 j. **Discrete source** – A radionuclide that has been processed so that its concentration within a
192 material has been purposely increased for use for commercial, medical, or research activities.

193

194 k. **Dose or radiation dose** – A generic term that means absorbed dose, dose equivalent,
195 effective dose equivalent, committed dose equivalent, committed effective dose equivalent,
196 or total effective dose equivalent, as defined in other paragraphs of this section.

197

198 l. **Dose equivalent (HT)** – The product of the absorbed dose in tissue, quality factor, and all
199 other necessary modifying factors at the location of interest. The units of dose equivalent are
200 the rem and sievert (Sv).

201

202 m. **Effective dose equivalent (HE)** – The sum of the products of the dose equivalent to the
203 organ or tissue (HT) and the weighting factors (WT) applicable to each of the body organs or
204 tissues that are irradiated ($HE = \sum WTHT$).

205

206 n. **Embryo/fetus** – The developing human organism from conception until the time of birth.

207

208 o. **Exposure** – Being exposed to ionizing radiation or to radioactive material.

209

210 p. **Extremity** - Hand, elbow, arm below the elbow, foot, knee, or leg below the knee.

211

212 q. **Gray** – The SI unit of absorbed dose. One gray is equal to an absorbed dose of 1
213 Joule/kilogram (100 rads).

214

215 r. **High radiation area** – An area, accessible to individuals, in which radiation levels from
216 radiation sources external to the body could result in an individual receiving a dose
217 equivalent in excess of 0.1 rem (1 mSv) in 1 hour at 30 centimeters from the radiation source
218 or 30 centimeters from any surface that the radiation penetrates.

219

220 s. **Individual** – Any human being.

221

222 t. **Individual monitoring** –

223 (1) The assessment of dose equivalent by the use of devices designed to be worn by an
224 individual;

225

- 226 (2) The assessment of committed effective dose equivalent by bioassay (see Bioassay) or by
227 determination of the time-weighted air concentrations to which an individual has been
228 exposed, i.e., DAC-hours; or
229
- 230 (3) The assessment of dose equivalent by the use of survey data.
231
- 232 u. **Individual monitoring devices** (individual monitoring equipment) - Devices designed to be
233 worn by a single individual for the assessment of dose equivalent such as film badges,
234 thermoluminescence dosimeters (TLDs), pocket ionization chambers, and personal ("lapel")
235 air sampling devices.
236
- 237 v. **Internal dose** – That portion of the dose equivalent received from radioactive material taken
238 into the body.
239
- 240 w. **Ionizing Radiation Safety Committee (IRSC)** – A body chartered by the NIST Director to
241 assist the Director and RSO in managing the radiation safety program.
242
- 243 x. **License** – A license issued under the regulations in parts 30 through 36, 39, 40, 50, 60, 61,
244 63, 70, or 72 of this chapter.
245
- 246 y. **Licensed material** - Source material, special nuclear material, or byproduct material
247 received, possessed, used, transferred or disposed of under a general or specific license
248 issued by the Commission.
249
- 250 z. **Licensee** – The holder of a license.
251
- 252 aa. **Limits** (dose limits) – The permissible upper bounds of radiation doses.
253
- 254 bb. **Member of the public** – Any individual except when that individual is receiving an
255 occupational dose.
256
- 257 cc. **Minor** – An individual less than 18 years of age.
258
- 259 dd. **Monitoring** (radiation monitoring, radiation protection monitoring) – The measurement of
260 radiation levels, concentrations, surface area concentrations or quantities of radioactive
261 material and the use of the results of these measurements to evaluate potential exposures and
262 doses.
263
- 264 ee. **NRC** – The Nuclear Regulatory Commission or its duly authorized representatives.

- 265 ff. **Occupational dose** – The dose received by an individual in the course of employment in
266 which the individual's assigned duties involve exposure to radiation or to radioactive material
267 from licensed and unlicensed sources of radiation, whether in the possession of the licensee
268 or other person. Occupational dose does not include doses received from background
269 radiation, from any medical administration the individual has received, from exposure to
270 individuals administered radioactive material and released under § 35.75, from voluntary
271 participation in medical research programs, or as a member of the public.
272
- 273 gg. **Person** –
- 274 (1) Any individual, corporation, partnership, firm, association, trust, estate, public or private
275 institution, group, Government agency other than the Commission or the Department of
276 Energy (except that the Department shall be considered a person within the meaning of
277 the regulations in 10 CFR chapter I to the extent that its facilities and activities are
278 subject to the licensing and related regulatory authority of the Commission under section
279 202 of the Energy Reorganization Act of 1974 (88 Stat. 1244), the Uranium Mill Tailings
280 Radiation Control Act of 1978 (92 Stat. 3021), the Nuclear Waste Policy Act of 1982 (96
281 Stat. 2201), and section 3(b)(2) of the Low-Level Radioactive Waste Policy Amendments
282 Act of 1985 (99 Stat. 1842)), any State or any political subdivision of or any political
283 entity within a State, any foreign government or nation or any political subdivision of any
284 such government or nation, or other entity; and
285
- 286 (2) Any legal successor, representative, agent, or agency of the foregoing.
287
- 288 hh. **Planned special exposure** – An infrequent exposure to radiation, separate from and in
289 addition to the annual dose limits.
290
- 291 ii. **Public dose** – The dose received by a member of the public from exposure to radiation or to
292 radioactive material released by a licensee, or to any other source of radiation under the
293 control of a licensee. Public dose does not include occupational dose or doses received from
294 background radiation, from any medical administration the individual has received, from
295 exposure to individuals administered radioactive material and released under § 35.75, or from
296 voluntary participation in medical research programs.
297
- 298 jj. **Rad** – The special unit of absorbed dose. One rad is equal to an absorbed dose of 100
299 ergs/gram or 0.01 joule/kilogram (0.01 gray).
300
- 301 kk. **Radiation** (ionizing radiation) – Alpha particles, beta particles, gamma rays, x-rays,
302 neutrons, high-speed electrons, high-speed protons, and other particles capable of producing

303 ions. Radiation, as used in this part, does not include non-ionizing radiation, such as radio- or
304 microwaves, or visible, infrared, or ultraviolet light.

305

306 ll. **Radiation area** – An area, accessible to individuals, in which radiation levels could result in
307 an individual receiving a dose equivalent in excess of 0.005 rem (0.05 mSv) in 1 hour at 30
308 centimeters from the radiation source or from any surface that the radiation penetrates.

309

310 mm. **Radiation Safety Officer** – An individual nominated by NIST and approved by the NRC
311 who is responsible for managing the radiation safety program. See NUREG [Series 1556](#) for
312 the minimum RSO qualifications.

313

314 nn. **Rem** – The special unit of any of the quantities expressed as dose equivalent. The dose
315 equivalent in rems is equal to the absorbed dose in rads multiplied by the quality factor (1
316 rem=0.01 sievert).

317

318 oo. **Residual radioactivity** – Radioactivity in structures, materials, soils, groundwater, and other
319 media at a site resulting from activities under the licensee's control. This includes
320 radioactivity from all licensed and unlicensed sources used by the licensee, but excludes
321 background radiation. It also includes radioactive materials remaining at the site as a result of
322 routine or accidental releases of radioactive material at the site and previous burials at the
323 site, even if those burials were made in accordance with the provisions of 10 CFR part 20.

324

325 pp. **Responsible Person** – The individual who is assigned sole responsibility for the safe use and
326 handling of generally-licensed or non-specifically-licensed RAM regulated by the NRC.

327

328 qq. **Restricted area** – An area, access to which is limited by the licensee for the purpose of
329 protecting individuals against undue risks from exposure to radiation and radioactive
330 materials. Restricted area does not include areas used as residential quarters, but separate
331 rooms in a residential building may be set apart as a restricted area.

332

333 rr. **Sievert** – The SI unit of any of the quantities expressed as dose equivalent. The dose
334 equivalent in sieverts is equal to the absorbed dose in grays multiplied by the quality factor (1
335 Sv=100 rems).

336

337 ss. **Site boundary** – The line beyond which the land or property is not owned, leased, or
338 otherwise controlled by the licensee.

339

340 tt. **Source material** –

- 341 (1) Uranium or thorium or any combination of uranium and thorium in any physical or
342 chemical form; or
343
- 344 (2) Ores that contain, by weight, one-twentieth of 1 percent (0.05 percent), or more, of
345 uranium, thorium, or any combination of uranium and thorium. Source material does not
346 include special nuclear material.
347
- 348 uu. **Special nuclear material** –
- 349 (1) Plutonium, uranium-233, uranium enriched in the isotope 233 or in the isotope 235, and
350 any other material that the Commission, pursuant to the provisions of section 51 of the
351 Act, determines to be special nuclear material, but does not include source material; or
352
- 353 (2) Any material artificially enriched by any of the foregoing but does not include source
354 material.
355
- 356 vv. **Supervised User (SU)** – An AU-approved individual who is authorized to use an AU’s
357 specifically-licensed RAM. As the name implies, the SU is under the supervision of the AU
358 who is responsible for the particular RAM.
359
- 360 ww. **Survey** – An evaluation of the radiological conditions and potential hazards incident to the
361 production, use, transfer, release, disposal, or presence of radioactive material or other
362 sources of radiation. When appropriate, such an evaluation includes a physical survey of the
363 location of radioactive material and measurements or calculations of levels of radiation, or
364 concentrations or quantities of radioactive material present.
365
- 366 xx. **Total Effective Dose Equivalent (TEDE)** – The sum of the effective dose equivalent (for
367 external exposures) and the committed effective dose equivalent (for internal exposures).
368
- 369 yy. **Unrestricted area** – An area, access to which is neither limited nor controlled by the
370 licensee.
371
- 372 zz. **Waste** – Those low-level radioactive wastes containing source, special nuclear, or byproduct
373 material that are acceptable for disposal in a land disposal facility. For the purposes of this
374 definition, low-level radioactive waste means radioactive waste not classified as high-level
375 radioactive waste, transuranic waste, spent nuclear fuel, or byproduct material as defined in
376 paragraphs (2), (3), and (4) of the definition of Byproduct material set forth in this section.
377
- 378 aaa. **Week** – Seven (7) consecutive days starting on Sunday.
379

380 bbb. **Whole body** - For purposes of external exposure, head, trunk (including male gonads), arms
381 above the elbow, or legs above the knee.

382

383 ccc. **Year** – The period of time beginning in January used to determine compliance with the
384 provisions of this part. The licensee may change the starting date of the year used to
385 determine compliance by the licensee provided that the change is made at the beginning of
386 the year and that no day is omitted or duplicated in consecutive years.

387

388

389 7. **ACRONYMS**

390 a. ALARA – As Low As is Reasonably Achievable.

391

392 b. ANSI – American National Standards Institute.

393

394 c. AU – Authorized User.

395

396 d. CDE – Committed Dose Equivalent.

397

398 e. CFR – Code of Federal Regulations.

399

400 f. DOT – Department of Transportation.

401

402 g. IRSC – Ionizing Radiation Safety Committee.

403

404 h. LSC – Liquid Scintillation Counter.

405

406 i. NCRP - National Council on Radiation Protection and Measurements.

407

408 j. NIST – National Institute of Standards and Technology.

409

410 k. NRC - Nuclear Regulatory Commission.

411

412 l. NUREG – NRC Technical Report Designation (Nuclear Regulatory Commission).

413

414 m. NVLAP - National Voluntary Laboratory Accreditation Program.

415

416 n. RAM – Radioactive material.

417

418 o. Reg. Guide – NRC Regulatory Guide.

- 419 p. RSO – Radiation Safety Officer.
420
421 q. SI – Système International d'Unités.
422
423 r. SU – Supervised User.
424
425 s. TEDE – Total Effective Dose Equivalent.
426
427

428 **8. REQUIREMENTS**

429 NIST is required as a licensee to develop, document, and implement a radiation protection
430 program commensurate with the scope and extent of specifically licensed activities and sufficient
431 to ensure compliance with the provisions of 10 CFR 20.
432

433 To ensure compliance NIST shall use, to the extent practical, engineering controls and
434 procedures based upon sound radiation protection principles to achieve occupational doses and
435 doses to members of the public that are ALARA.
436

437 Except as specifically provided otherwise in NRC Materials License 05-03166-06, NIST shall
438 conduct its program in accordance with the statements, representations, and procedures
439 contained in the license. The U.S. NRC's regulations shall govern unless the statements,
440 representations, and procedures in the license application and correspondence are more
441 restrictive than the regulations. [LC 20]
442

443 **SPECIAL CIRCUMSTANCES:**

- 444
- 445 **1) RAM that is not specifically-licensed is not typically subject to a specific license or**
446 **the requirements of a written radiation protection program unless required by**
447 **regulation.**
448
 - 449 **2) Therefore, RAM (e.g., generally-licensed material of devices, “exempt quantities” of**
450 **byproduct material, small quantities of source material, etc.) not listed on a specific**
451 **license is managed on a case-by-case basis in accordance with applicable regulations**
452 **and is only subject to the requirements in this suborder when specifically applicable.**
453
 - 454 **3) Responsible Persons designated by official first-level supervisors shall ensure**
455 **regulatory and program compliance for their RAM that is not specifically-licensed,**
456 **inform the RSO of receipt, use, and storage locations, and transfer the RAM in**

457 **accordance with regulatory requirements. If disposal is permitted, it is to be**
458 **transferred to the RSO for proper disposal.**

459
460 The following elements are required in this program and are implemented through various
461 documents (e.g., radiation safety manual, procedures, instructions, guideline, etc.). Each element
462 is addressed separately below:

- 463
- 464 a. Radiation Safety Training;
 - 465
 - 466 b. Radiation Monitoring Instruments;
 - 467
 - 468 c. Radiological Surveys;
 - 469
 - 470 d. Material Receipt, Accountability, and Security;
 - 471
 - 472 e. Safe Use of Radionuclides and Emergency Procedures;
 - 473
 - 474 f. Occupational Dose;
 - 475
 - 476 g. Transportation;
 - 477
 - 478 h. Waste Management;
 - 479
 - 480 i. Program Review;
 - 481
 - 482 j. Notifications and Reports; and
 - 483
 - 484 k. Recordkeeping.

485

486 a. Radiation Safety Training

487

488 (1) Qualified Instructors [**LC 20A Item 8C**]

489

- 490 (a) The RSO and Authorized Users (AUs) are qualified by default for providing radiation
- 491 safety training.

492

- 493 (b) Other instructors may be approved at the discretion of the RSO.

494

495 (2) Acceptable Training Methods [**LC 20A Item 8C**]

- 496 (a) Attending qualified instructor training sessions;
497
498 (b) Reading materials provided by qualified instructors;
499
500 (c) Computer-based training approved by a qualified instructor;
501
502 (d) Watching Radiation Safety video presentations approved by a qualified instructor;
503
504 (e) Attending training provided by other NIST staff or outside vendors approved by the
505 RSO or an AU;
506
507 (f) Attending radiation safety related seminars as approved by the RSO or an AU; or
508
509 (g) Other means at the discretion of the RSO or an AU.
510
- 511 (3) Notices, Instructions, and Reports to Workers (10 CFR 19) Training [**LC 20A Item 8B**]
512
- 513 (a) All individuals who in the course of employment are likely* to receive, in a year, an
514 occupational dose in excess of 100 mrem shall be instructed in the following in a
515 manner commensurate with potential radiological health protection problems present
516 in the workplace:
517
- 518 i. The storage, transfer, or use of radiation and/or radioactive material.
519
 - 520 ii. The health protection problems associated with exposure to radiation and/or
521 radioactive material, in precautions or procedures to minimize exposure, and
522 in the purposes and functions of protective devices employed.
523
 - 524 iii. The applicable provisions of NRC regulations and licenses for the protection
525 of personnel from exposure to radiation and/or radioactive material as well as
526 the expectation that the individual shall comply with said regulations and
527 license conditions.
 - 528 iv. Their responsibility to report promptly to the RSO any condition which may
529 lead to or cause a violation of NRC regulations and licenses, or potential
530 unnecessary exposure to radiation and/or radioactive material.
531
 - 532 v. The appropriate response to warnings made in the event of any unusual
533 occurrence or malfunction that may involve exposure to radiation and/or
534 radioactive material.

535 vi. The radiation exposure reports which workers may request pursuant to 10
536 CFR 19.13.

537
538 *NOTE: When determining the likelihood of occurrence, consider assigned activities
539 during normal and abnormal situations involving exposure to radiation and/or
540 radioactive material which can reasonably be expected to occur during the life of a
541 licensed facility.

542
543 (4) Content of Training [LC 20A Item 8C]

544
545 (a) Training shall be of sufficient content and length to ensure the compliant, safe use
546 and handling of RAM. [LC 20A Item 8C]

547
548 (b) NUREG 1556 Volume 7, Appendix J should be used as a guide for selecting training
549 topics.

550
551 (c) Training should include the following elements as applicable:

- 552
- 553 i. Radiation Protection Principles;
 - 554 (i) ALARA Policy and Practices;
 - 555 ii. Characteristics of Ionizing Radiation;
 - 556 iii. Units of Radiation Dose and Quantities;
 - 557 iv. Proper Selection and Use of Radiation Detection Instrumentation;
 - 558 v. Biological Hazards of Exposure to Radiation (with respect to the scope of
559 source use):
 - 560 (i) Declared Pregnant Woman Policy; and
 - 561 (ii) Minors in NIST Laboratories Policy;
 - 562 vi. Acquisition and Disposition of RAM;
 - 563 vii. Accountability and Control of RAM;
- 564
565
566
567
568
569
570
571
572
573

- 574 (i) Posting and Labeling of RAM;
575
576 viii. Hands-on Use of Radioactive Material;
577
578 ix. Emergency Response; and
579
580 x. Dosimetry.
581
- 582 (d) Ancillary Personnel may not need extensive training, however, the following is the
583 minimum they shall receive:
584
- 585 i. Radiation hazards they may be exposed to:
586
587 (i) Declared Pregnant Woman Policy; and
588
589 (ii) Minors in NIST Laboratories Policy;
590
- 591 ii. Response to Radiological Postings;
592
- 593 iii. The appropriate precautions necessary to keep exposure ALARA;
594
- 595 iv. Acquisition and Disposal of RAM;
596
- 597 v. Emergency Response; and
598
- 599 vi. Recognition of events which requires informing the RSO.
600
- 601 (e) General Employee & Associate Orientation
602
- 603 i. Overview of radiation safety policies and procedures:
604
605 (i) ALARA;
606
607 (ii) Recognition and Response to Radiological Postings; and
608
609 (iii) Emergency Response;
610
- 611 ii. Acquisition and Disposal of RAM.
612

613 (5) Assessment of Training [LC 20A Item 8C]
614

615 (a) Training shall be assessed by written or verbal testing, except in the case of the RSO
616 being self-taught, in which no assessment will be performed.
617

618 (b) A test will be considered successfully completed with a score of at least 80%.
619

620 (c) Additional assessments may be used to demonstrate proficiency in practical
621 applications (e.g., instrument use in accordance with operating manual).
622

623 (6) Frequency of Training for Personnel [LC 20A Item 8C]
624

625 (a) Before assuming duties with, or in the vicinity of, radioactive materials or radiation.
626

627 (b) Conditional training circumstances at the discretion of the RSO or an AU.
628

629 i. Whenever there is a significant, applicable change in duties, regulations, or
630 the terms of the license.
631

632 ii. Behavioral or “for-cause” retraining.
633

634 (c) Refresher Training.
635

636 i. Annually:
637

638 (i) Authorized Users; and
639

640 (ii) Responsible Persons.
641

642 ii. Biennially
643

644 (i) Supervised Users (SUs); and
645

646 (ii) Ancillary Personnel.
647

648 b. Radiation Monitoring Instruments

649 (1) NIST will use instruments that meet the radiation monitoring instrument specifications
650 published in Appendix M to NUREG 1556, Vol. 7 dated December 1999. [LC 20A Item
651 10A]

- 652 (2) At a minimum, NIST shall possess, or have access to, the following or equivalent [LC
653 **20A Item 10A]**:
- 654 (a) An ion chamber dose rate meter;
- 655 (b) A pancake Geiger Mueller detector calibrated in counts per minute;
- 656 (c) An alpha detector calibrated in counts per minute; and
- 657 (d) A Liquid Scintillation Counter (LSC).
- 658
- 659 (3) Instruments and equipment used for quantitative radiation measurements (*e.g.*, dose rate
660 and effluent monitoring) shall be calibrated periodically for the radiation measured in
661 accordance with ANSI N323-1978.
- 662
- 663 c. Radiological Surveys
- 664 (1) NIST shall make or cause to be made, surveys of areas, including the subsurface, that:
- 665 (a) May be necessary for compliance with the regulations in 10 CFR 20; and
- 666 (b) Are reasonable under the circumstances to evaluate:
- 667 i. The magnitude and extent of radiation levels;
- 668 ii. Concentrations or quantities of residual radioactivity; and
- 669 iii. The potential radiological hazards of the radiation levels and residual
670 radioactivity detected.
- 671
- 672 (2) NIST shall make or cause to be made, as appropriate, surveys of radiation levels in
673 unrestricted and controlled areas and radioactive materials in effluents released to
674 unrestricted and controlled areas to demonstrate compliance with the dose limits for
675 individual members of the public in 10 CFR 20.1301.
- 676
- 677 (3) NIST shall survey the facility and maintain contamination levels in accordance with the
678 survey frequencies and contamination levels published in Appendix Q to NUREG 1556,
679 Vol. 7, dated December 1999. [LC 20A Item 10E]
- 680
- 681
- 682
- 683
- 684
- 685
- 686
- 687
- 688
- 689

- 690 (4) Sealed sources and detector cells shall be tested for leakage and/or contamination at
691 intervals not to exceed 6 months or at such intervals specified in the certificate of
692 registration issued by the U.S. NRC under 10 CFR 32.210 or by an Agreement State. [LC
693 14A]
694
- 695 (5) Sealed sources and detector cells designed to emit alpha particles shall be tested at
696 intervals not to exceed 3 months. [LC 14B]
697
- 698 (6) Sealed sources and detector cells shall be tested for leakage prior to use unless
699 accompanied by a satisfactory leak test by the previous owner in the last 6 months (3
700 months for primarily alpha emitters) [LC 14C]
701
- 702 (7) The following sealed sources and detector cells need not be tested [LC 14D]
703
- 704 (a) Those containing 100 μCi (3.7 MBq) or less of beta- and/or gamma-emitting
705 radioactive material;
 - 706
 - 707 (b) Those containing 10 μCi (370 KBq) or less of alpha-emitting radioactive material;
708
 - 709 (c) Those containing only H-3 (tritium);
710
 - 711 (d) Those containing only material with a half-life of 30 days or less; or
712
 - 713 (e) Those containing only gaseous radioactive material.
714
- 715 (8) Sealed sources need not be leak tested if they are in storage **AND** are not in use.
716 However, when the source is removed from storage for use or transfer and has not been
717 tested within the required interval, the source shall be leak tested prior to use or transfer.
718 [LC 14E]
719
- 720 (9) No sealed source shall be stored for a period of more than 10 years without being tested
721 for leakage. [LC 14E]
722
- 723 (10) The leak test shall be capable of detecting the presence of 0.005 μCi (185 Bq) of
724 radioactive material on the test sample. [LC 14F]
725
- 726 (11) Tests for leakage and/or contamination, including leak test sample collection and
727 analysis, shall be performed by NIST or by other persons specifically licensed by the
728 U.S. NRC or an Agreement State to perform such a service. [LC 14G]

729 d. Material Receipt, Accountability, & Security

730
731
732
733
734
735
736
737
738
739
740
741
742
743
744
745
746
747
748
749
750
751

- (1) Possession and use of specifically-licensed material is limited to the license conditions of license 05-03166-06. **[LC 6, 7, 8, & 9]**
- (2) Possession and use of generally licensed material is limited to the applicable general license.
- (3) Possession and use of non-specifically licensed material is limited to the applicable regulation(s).
- (4) NIST shall use written procedures for ensuring that material is properly received and opened. **[LC 20A Item 10B]**
- (5) Packages (if identified as radioactive material) shall be inspected by receiving personnel and secured in a locked enclosure reserved for radioactive material until they are retrieved by the RSO for monitoring. **[LC 20A Item 9A]**
- (6) NIST shall ensure monitoring of the external surfaces of a package containing licensed material as indicated in the table below (presumes package is received during normal working hours. If received outside of normal working hours, monitor the package within 3 hours of the beginning of the next work day):

Package	Contents	Survey Type	Survey Time*
Labeled (White I, Yellow-II, Yellow-III)	Gas or Special Form, Greater Than Type A Quantity	Radiation Level	As soon as practicable, but not later than 3 hours after receipt of the package
Labeled (White I, Yellow-II, Yellow-III)	NOT Gas or Special Form, Greater Than Type A Quantity	Contamination & Radiation Level	As soon as practicable, but not later than 3 hours after receipt of the package
Labeled (White I, Yellow-II, Yellow-III)	Gas or Special Form, Less Than Type A Quantity	None	None
Labeled (White I, Yellow-II, Yellow-III)	NOT Gas or Special Form, Less Than Type A Quantity	Contamination	As soon as practicable, but not later than 3 hours after receipt of the package

Not Labeled	Licensed Material	None	None
Damaged	Licensed Material	Contamination & Radiation Level	As soon as practicable, but not later than 3 hours after receipt of the package

752

753 (7) NIST shall use written procedures for ensuring material accountability and security are
 754 implemented. **[LC 20A Item 10B]**

755

756 (8) Licensed material that is in a controlled or unrestricted area shall be stored and secured
 757 from unauthorized removal or access when not in use. **[LC 20A Item 9B]**

758

759 (9) Licensed material that is in use in a controlled or unrestricted area shall be controlled and
 760 under constant surveillance. **[LC20A Item 9C]**

761

762 (10) Physical inventories shall be conducted every 6 months, or at other intervals approved by
 763 the NRC, to account for all sealed sources and/or devices received and possessed under
 764 NRC Materials License No. 05-03166-06. **[LC 13 and 20A Item 10B]**

765

766 e. Safe Use of Radionuclides and Emergency Procedures

767

768 (1) NIST shall develop and implement procedures for the safe use of licensed material,
 769 including security of materials, and emergencies involving licensed material. **[LC 20A**
 770 **Item 10D]**

771

772 (2) Posting of notices to workers shall be in accordance with 10 CFR 19.11.

773

774 (3) Planned special exposures shall be in accordance with 10 CFR 20.1206.

775

776 (4) NIST shall conduct operations so that (1) The total effective dose equivalent to individual
 777 members of the public from the licensed operation does not exceed 0.1 rem (1 mSv) in a
 778 year, and (2) The dose in any unrestricted area from external sources does not exceed
 779 0.002 rem (0.02 mSv) in any one hour.

780

781 (a) If NIST permits members of the public to have access to controlled areas, the limits
 782 for members of the public continue to apply to those individuals.

783

784 (5) Specifically-licensed material shall be stored or used only at NIST facilities located at
 785 325 Broadway, Buildings 1, 23, and 81. **[LC 10]**

- 786 (6) Specifically-licensed material shall be used by, or under the supervision of, an
 787 Authorized User. [LC 11]
 788
 789 (7) Specifically-licensed material shall be used for the purposes approved by license 05-
 790 03166-06 [LC 9A-E and LC 20A]
 791
 792 (8) Sealed sources or detectors cells containing licensed material shall not be opened or
 793 sources removed from source holders. [LC 15]
 794
 795 (9) Licensed material shall not be used in or on human beings. [LC 16]
 796
 797 (10) NIST shall not offer licensed material for commercial distribution. [LC 17]
 798
 799 (11) NIST shall not use licensed material where activity is planned to be released to the
 800 environment. [LC 19]
 801

802 f. Occupational Dose

- 803 (1) NIST shall do a prospective evaluation and determine that unmonitored individuals are
 804 not likely to receive, in one year, a radiation dose in excess of 10% of the allowable
 805 limits in 10 CFR Part 20 or NIST will monitor individuals in accordance with the criteria
 806 in the section entitled 'Radiation Safety Program - Occupational Dose' in NUREG-1556,
 807 Vol. 7, dated December 1999. [LC 20A Item 10C]
 808
 809 (2) NIST shall implement radiation safety procedures which ensure doses are ALARA, but in
 810 any case do not exceed the following dose limits:
 811

Type of Exposure	Regulatory Limits
Whole body, TEDE	5000 mrem/year
Sum of DDE and CDE to organ	50000 mrem/year
Skin or Extremity	50000 mrem/year
Lens of Eye	15000 mrem/year
Internal	1.0 ALI/year 10 mg/week soluble uranium
Embryo/Fetus*	500 mrem/gestation period
Member of the Public	100 mrem/year

812
 813 *See Reg. Guide 8.13 and 8.29 for additional information.
 814

- 815 (3) All personnel dosimeters (except for direct and indirect reading pocket ionization
816 chambers and those dosimeters used to measure the dose to the extremities) that require
817 processing to determine the radiation dose and that are used by licensees to comply with
818 10 CFR 20 or with conditions specified in a license must be processed and evaluated by a
819 dosimetry processor:
- 820
- 821 (a) Holding current personnel dosimetry accreditation from the National Voluntary
822 Laboratory Accreditation Program (NVLAP) of the National Institute of Standards
823 and Technology; and
824
- 825 (b) Approved in this accreditation process for the type of radiation or radiations included
826 in the NVLAP program that most closely approximates the type of radiation or
827 radiations for which the individual wearing the dosimeter is monitored.
828
- 829 g. Transportation
- 830 (1) NIST may transport licensed material only in accordance with the provisions of 10 CFR
831 71, "Packaging and Transportation of Radioactive Material". [LC 18]
832
- 833 h. Waste Management
- 834 (1) NIST will use the model waste procedures published in Appendix T to NUREG 1556,
835 Vol. 7, dated December 1999 to manage waste disposition. [LC 20A Item 11]
836
- 837 (2) NIST shall use contamination minimization techniques to reduce the generation of
838 radioactive waste.
839
- 840 i. Program Reviews (Audits)
- 841 (1) The licensee shall periodically (at least annually) review (audit) the radiation protection
842 program content and implementation to ensure the following:
843
- 844 (a) Compliance with NRC and DOT regulations (as applicable), and the terms and
845 conditions of the license; and
846
- 847 (b) Occupational doses and doses to members of the public are ALARA.
848
- 849 (2) NIST should use the model program review forms published in Appendix H to NUREG
850 1556, Vol. 7, Revision 1, dated February 2018, to perform program reviews. However, if
851 the forms are not sufficient for a particular review it may be modified in full or part.
852

- 853 j. Notifications and Reports
854 (1) Radiation exposure data for an individual, and the results of any measurements, analyses,
855 and calculations of radioactive material deposited or retained in the body of an individual,
856 shall be reported to the individual as specified in 10 CFR 19.13.
857
858 (2) NIST shall inform an individual, in writing, of the dose from a planned special exposure
859 within 30 days of the exposure as specified in 10 CFR 20.1206 and 20.2204.
860
861 (3) NIST shall submit a written report to the Administrator of the Region 4 NRC Office
862 within 30 days following any planned special exposure, informing the NRC that a
863 planned special exposure was conducted and indicating the date the planned special
864 exposure occurred and the information required by 10 CFR 20.2105.
865
866 (4) If a leak test reveals the presence of 0.005 μCi (185 Bq) or more of removable
867 contamination [LC 14F]:
868
869 (a) The source shall be immediately removed from service; and
870
871 (b) A report shall be filed with the NRC in accordance with 10 CFR 30.50(c)(2).
872
- 873 k. Recordkeeping
874 (1) NIST shall use the units: curie, rad, rem, including multiples and subdivisions, and shall
875 clearly indicate the units of all quantities on records required by this program.
876
877 (a) NIST may include SI units in parentheses following the standard units.
878
879 (b) Shipping manifests for waste are an exception in that SI units are required to be used;
880 the standard units above may follow in parentheses.
881
882 (2) All records must be legible throughout the specified retention period.
883
884 (3) Records of the radiation safety program, including provisions of the program, shall be
885 retained until the NRC terminates the license requiring the records.
886
887 (4) Records of audits and other reviews of program content and implementation shall be
888 retained for 3 years after the record is made.
889
890 (5) Records showing the results of surveys and calibrations shall be retained for 3 years after
891 the record is made.

- 892 (6) Records showing the transfer of licensed material shall be retained for 3 years after the
893 transfer date.
- 894
- 895 (7) The following records are to be retained until the NRC terminates the license requiring
896 the records:
- 897
- 898 (a) Records of the results of surveys to determine the dose from external sources and
899 used, in the absence of or in combination with individual monitoring data, in the
900 assessment of individual dose equivalents;
- 901
- 902 (b) Records of the results of measurements and calculations used to determine individual
903 intakes of radioactive material and used in the assessment of internal dose;
- 904
- 905 (c) Records showing the results of air sampling, surveys, and bioassays;
- 906
- 907 (d) Records of the results of measurements and calculations used to evaluate the release
908 of radioactive effluents to the environment; and
- 909
- 910 (e) Records sufficient to demonstrate compliance with the dose limit for individual
911 members of the public.
- 912
- 913 (8) Records of the conduct of a planned special exposure shall be in accordance with 10 CFR
914 20.2105.
- 915
- 916 (9) Records of inventories shall be maintained for 5 years from the date of each inventory
917 and shall include the radionuclides, quantities, manufacturer's name and identifier such
918 as model number, and the date of the inventory. **[LC 13 & 20A Item 10B]**
- 919
- 920 (10) Records of leak test results shall be kept in units of μCi and shall be maintained for 3
921 years. **[LC 14H]**
- 922
- 923

924 **9. RESPONSIBILITIES**

925 Roles and responsibilities common to all NIST ionizing-radiation-safety suborders can be found
926 in **NIST Order 7201.00**.

927

928 ***NOTE:*** *A person may delegate the authority of their position to other individuals in order to*
929 *perform tasks that are necessary to meet program requirements unless a task is prohibited as*

930 *non-delegable. In any case, the person retains the responsibility (a.k.a. burden) for ensuring*
931 *the task is successfully accomplished.*

932

933 a. Line Management (as it applies within their subordinate organizational body)

934

935 (1) Managers are responsible for ensuring that resources are authorized and properly
936 allocated to sufficiently support radiological compliance and safety;

937

938 (2) Supervisors and managers are responsible for supporting the RSO, Responsible Persons,
939 and Users in those instances when it is necessary for those individuals to assert authority;
940 and

941

942 (3) Official first-level supervisors are responsible for designating Responsible Persons for
943 non-specifically-licensed RAM.

944

945 b. Radiation Safety Officer

946

947 (1) Ensuring compliance with NRC and Department of Transportation (DOT) regulations
948 and the conditions of material licenses by developing, managing, and enforcing a written
949 radiation safety program;

950

951 (2) Complying with, and ensuring, applicable NIST policies and other directives are
952 incorporated into the written radiation safety program;

953

954 (3) Providing health physics services within capabilities of the Boulder Safety, Health, and
955 Environment Division;

956

957 (4) Submitting NRC License Applications, Applications for License Amendment, responses
958 to Requests for Additional Information, Licensee Event Reports, and responses to
959 Notices of Violation to the IRSC to review for completeness and accuracy;

960

961 (5) Notifying the NRC of radiological incidents in accordance with the applicable
962 regulations; and

963

964 (6) Ensuring that the Ionizing Radiation Safety Committee (IRSC) is informed of the posting
965 of any RAM-related reports to the NIST Incident Reporting and Investigation System.

966

967

968 c. Authorized Users

- 969
- 970 (1) Supervising the use of the radioactive material they manage;
- 971
- 972 (2) Ensuring that radioactive materials used in his or her particular lab or area are used safely
- 973 and according to regulatory and program requirements;
- 974
- 975 (3) Ensuring that procedures and engineering controls are used to keep occupational doses
- 976 and doses to members of the public ALARA;
- 977
- 978 (4) Ensuring that they and their users have adequate and appropriate training to provide
- 979 reasonable assurance that they will use licensed material safely, including maintaining
- 980 security of, and access to, licensed material, and respond appropriately to events or
- 981 accidents involving licensed material; and
- 982
- 983 (5) Ensuring that ancillary personnel have adequate and appropriate training to provide
- 984 reasonable assurance that they will maintain their doses ALARA, follow posted
- 985 instructions, and respond appropriately to radiological events.
- 986
- 987 d. Supervised Users
- 988
- 989 (1) Following current approved radiation safety procedures.
- 990
- 991 e. Ancillary Personnel
- 992
- 993 (1) Following current approved radiation safety procedures.
- 994
- 995 f. Ionizing Radiation Safety Committee
- 996
- 997 (1) Reviewing NRC License Applications, Applications for License Amendment, responses
- 998 to Requests for Additional Information, Licensee Event Reports, and responses to
- 999 Notices of Violation for completeness and accuracy, and advise the NIST Director as
- 1000 necessary, prior to their submittal to the NRC;
- 1001
- 1002 (2) For the following types of events, reviewing the adequacy of the investigations and
- 1003 actions to prevent recurrence, and reporting to the NIST Director on their completion:
- 1004
- 1005 (a) NRC-reportable occurrences involving radioactive material;
- 1006
- 1007 (b) NRC-identified violations of radiation safety program requirements;

- 1008
1009 (c) Self-identified apparent violations of radiation safety program requirements that
1010 could be characterized by the NRC as Severity Level I, II, or III; and
1011
1012 (d) Any events identified to the IRSC by the RSO that have, or may have, adverse
1013 impacts on ALARA, radiation safety, or regulatory compliance; and
1014
1015 (3) Reviewing the results of program reviews (audits) and tracking and reporting to the NIST
1016 Director on the resolution of all reported findings and apparent violations; and
1017
1018 (4) Assisting the RSO upon request from the RSO.
1019

1020 g. Responsible Persons

- 1021 (1) Supervising the use of the radioactive material they manage;
1022
1023 (2) Ensuring that radioactive material used in their particular lab or area are used safely and
1024 according to regulatory and applicable program requirements;
1025
1026 (3) Ensuring that procedures and engineering controls are used to keep occupational doses
1027 and doses to members of the public ALARA;
1028
1029 (4) Ensuring that they and their users have adequate and appropriate training to provide
1030 reasonable assurance that they will use licensed material safely, including maintaining
1031 security of, and access to, licensed material, and respond appropriately to events or
1032 accidents involving licensed material; and
1033
1034 (5) Ensuring that ancillary personnel have adequate and appropriate training to provide
1035 reasonable assurance that they will maintain their doses ALARA, follow posted
1036 instructions, and respond appropriately to radiological events.
1037
1038

1039 **10. AUTHORITIES & ACCOUNTABILITIES**

1040 *“The key to an effective [radiation protection] program is the formal delegation of authority to*
1041 *competent staff members.” – NCRP Report No. 127*

1042

1043 a. Special Delegation of Authority

1044

1045 (1) The RSO, AU, SU, and Responsible Person roles are delegated the authority of the NIST
1046 Director to ensure the safe and compliant use of RAM. This power *is not* limited by
1047 organizational boundaries, roles, positions, or employment statuses.

1048
1049 (a) Line management will support these roles in those instances where it is necessary for
1050 them to assert authority.

1051
1052 i. If any radiological dispute arises, the RSO is the final arbiter.

1053
1054 b. Accountability

1055
1056 (1) Each individual fulfilling a role in this program is accountable for their conduct.
1057 Depending on the circumstances, the consequences for inappropriate behavior may
1058 progressively escalate (e.g., peer coaching, re-training, suspension or revocation of RAM
1059 use).

1060
1061 (a) Personnel actions may be initiated by line management.

1062
1063 (b) Civil or criminal actions may be taken depending on circumstances.

1064
1065
1066 **11. DIRECTIVE OWNER**

1067 a. Chief Safety Officer

1068
1069
1070 **12. APPENDICES**

1071 A. Revision History

1072

1073
1074

Appendix A. Revision History

Revision No.	Approval Date	Deployment Start Date	Effective Date	Brief Description of Change; Rationale
0	02/10/17	NA	02/10/17	None – Initial document
1	04/17/18	NA	04/17/18	<ul style="list-style-type: none">Added responsibility for the IRSC to review the results of internal and external audits of the Boulder radioactive materials program and tracking and reporting to the NIST Director on the resolution of all reported findings and apparent violations;

1075